

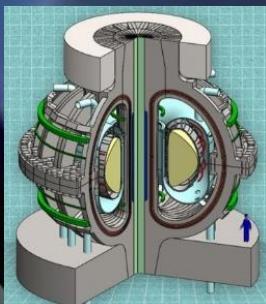


# 核融合マグネットへの適用をめざした 大電流高温超伝導導体の開発



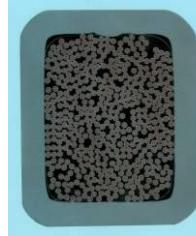
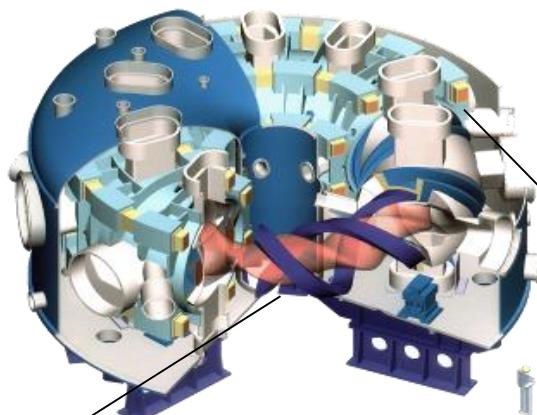
核融合科学研究所

柳 長門

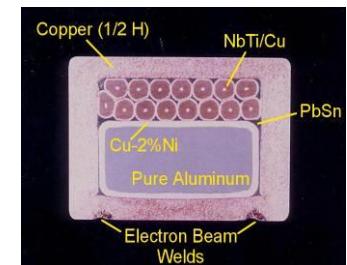


第16回 QUEST 研究会  
2019年10月4日  
九州大学 応用力学研究所

# *From LHD to FFHR*



**Poloidal Coils  
NbTi-CICC (world first)**



**Helical Coils  
NbTi  
pool-boiling**

**Construction : 1990-1998  
Operation : 1998-**



***What kind of superconductor should be used?***

## SC Material Selection

**1. LTS → Nb<sub>3</sub>Al, Nb<sub>3</sub>Sn**

**2. HTS → YBCO**

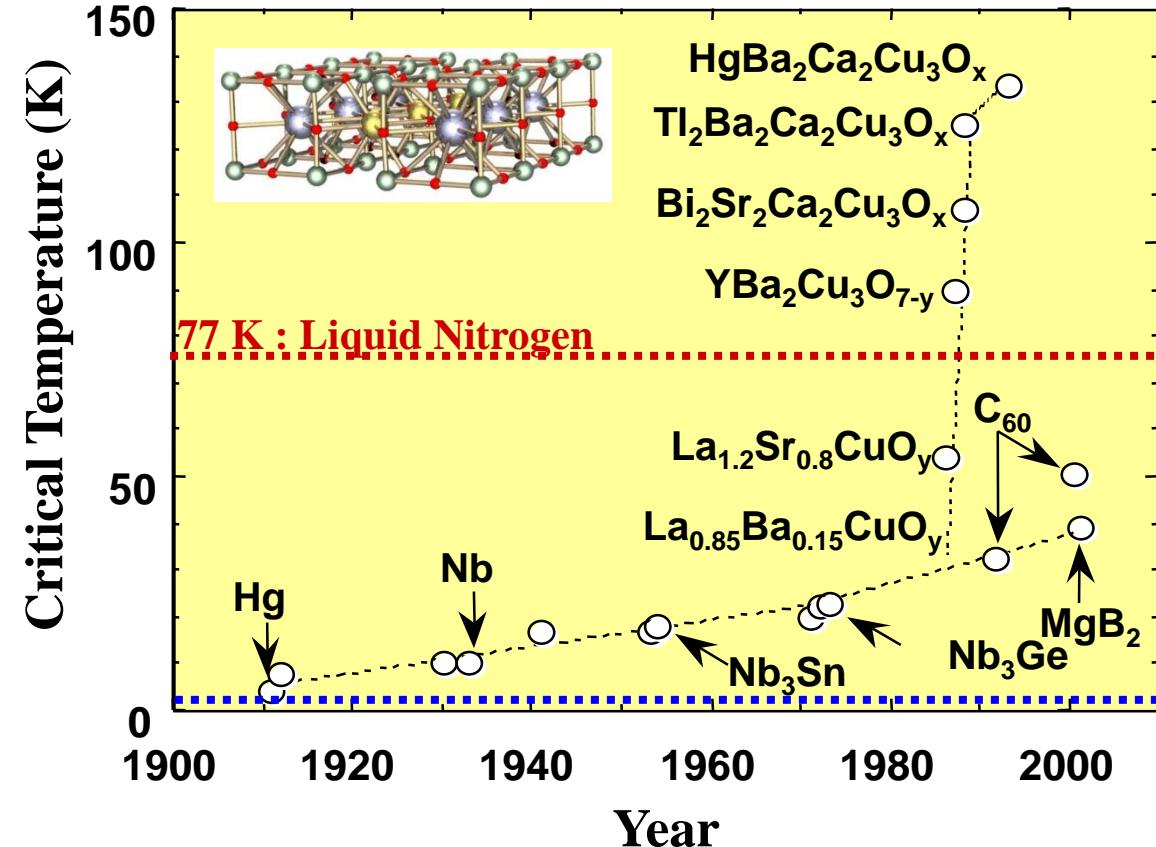
## Conductor Selection

**1. Force-cooled LTS-CIC conductor**

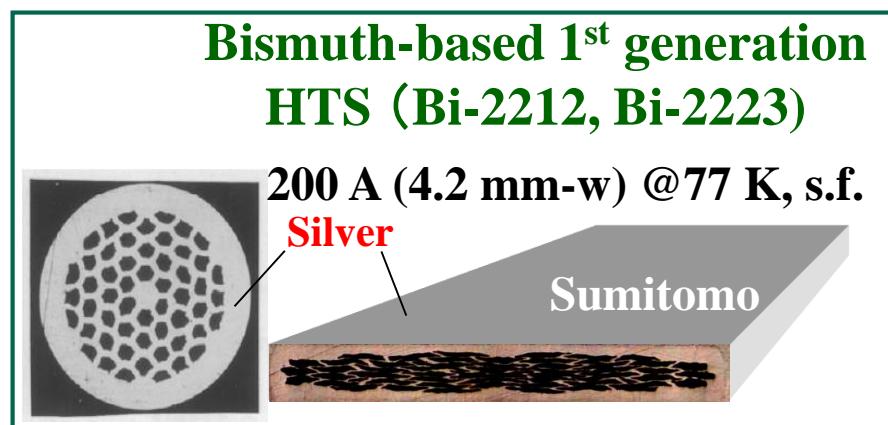
**2. Indirectly-cooled LTS conductor**

**3. Helium gas cooled HTS conductor**

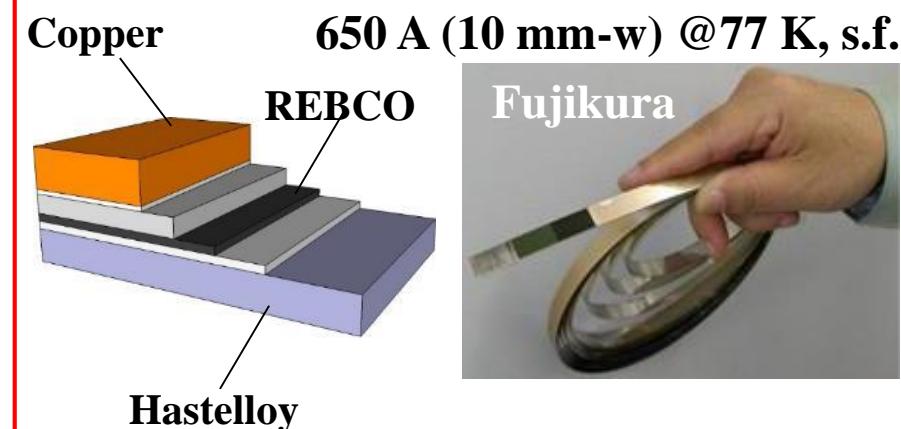
# High-Temperature Superconductors (HTS)



- Discovery of superconductivity of copper-oxide materials in 1986
- Tremendous progress of wire (**tape**) production technology
- Applicable to semi-conductor production, power cable, motor, transformer, MRI, medical accelerator, MAGLEV, SMES
- What about for fusion magnets?*



## Rare Earth-based 2<sup>nd</sup> generation HTS (YBCO, GdBCO)

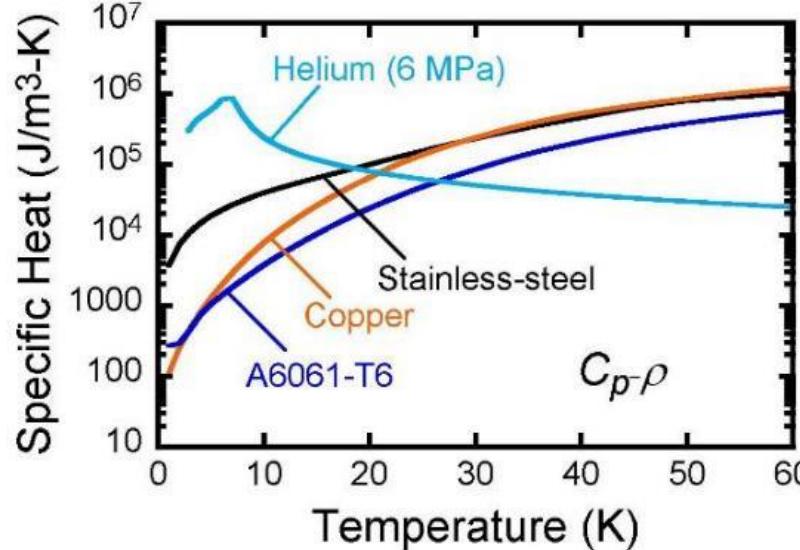
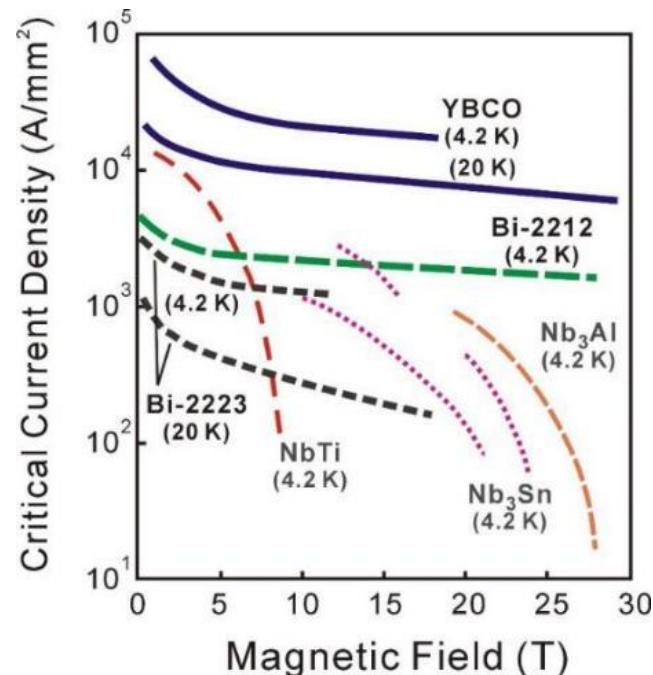


# High-Temperature Superconductor (HTS)

- (1) High critical current to high field
- (2) High cryogenic stability
- (3) Low cryogenic power
- (4) High mechanical rigidity
- (5) Industrial production of tapes
- (6) Saving helium resources



- High field
- High temp.
- High heat



Stability Margin

$$\Delta Q < C_p \rho \Delta T$$

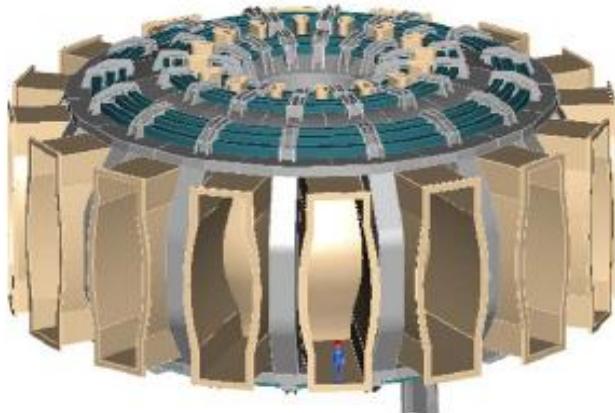
$$C_p \rho \Delta T \approx 2 \times 10^5 \text{ (J/m}^3\text{K}) \times 10 \text{ (K)}$$

$$\approx 2 \text{ (J/cc)}$$

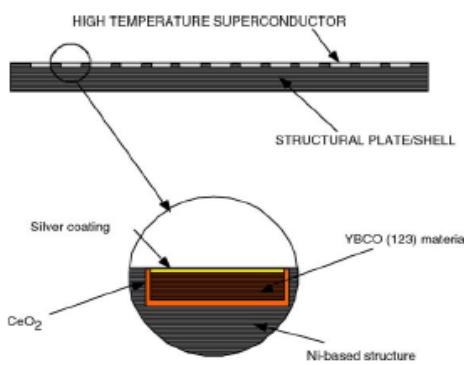
Higher than CIC conductor  
→ Low quench risk!

N. Yanagi, S. Ito, et al.,  
Plasma and Fusion Research  
9 (2014) 1405013

# Pioneering Work of applying HTS to tokamak reactor designs

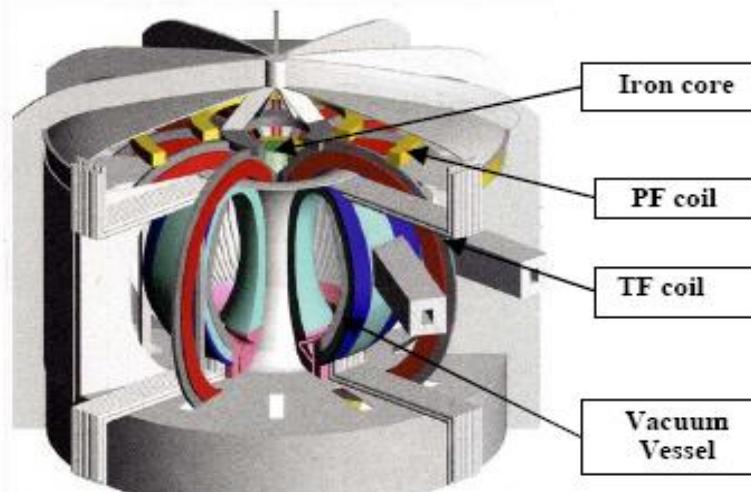


**ARIES-AT (USA)**

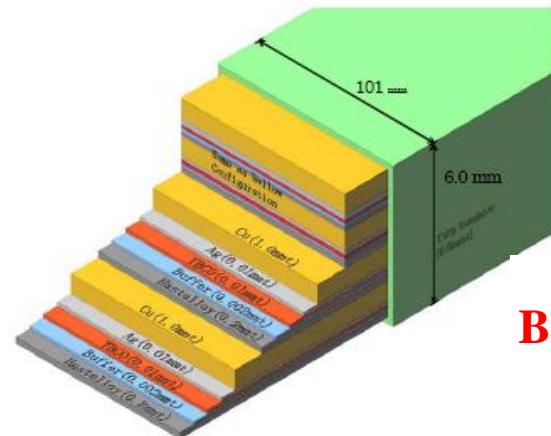


**YBCO**

F. Dahlgren *et al.*  
(2006)

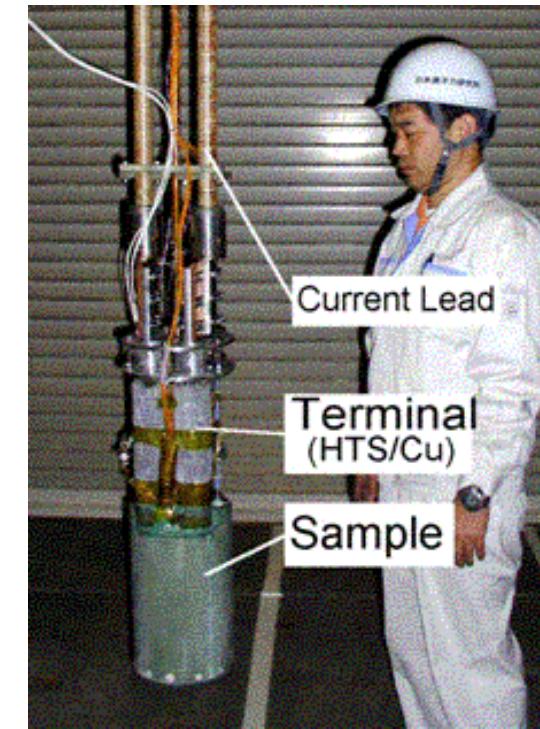


**VECTOR (JAEA)**

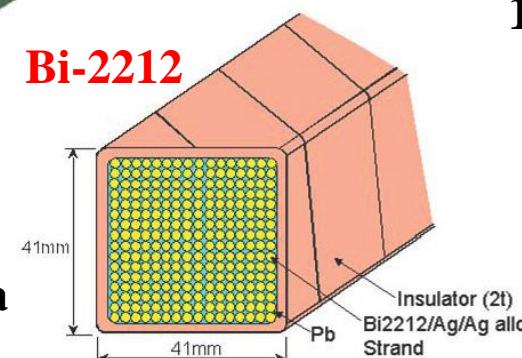


**YBCO**

T. Ando, S. Nishio, H. Yoshimura  
(2004)



**Bi-2212 CIC conductor**  
10 kA@20 K, 12 T  
T. Isono *et al.*  
(2003)



# *Pioneering Work of applying HTS to helical reactor designs*

H. Hashizume, S. Kitajima, S. Ito, K. Yagi, Y. Usui, Y. Hida, A. Sagara

“Advanced Fusion Reactor Design using Remountable HTc SC Magnet”

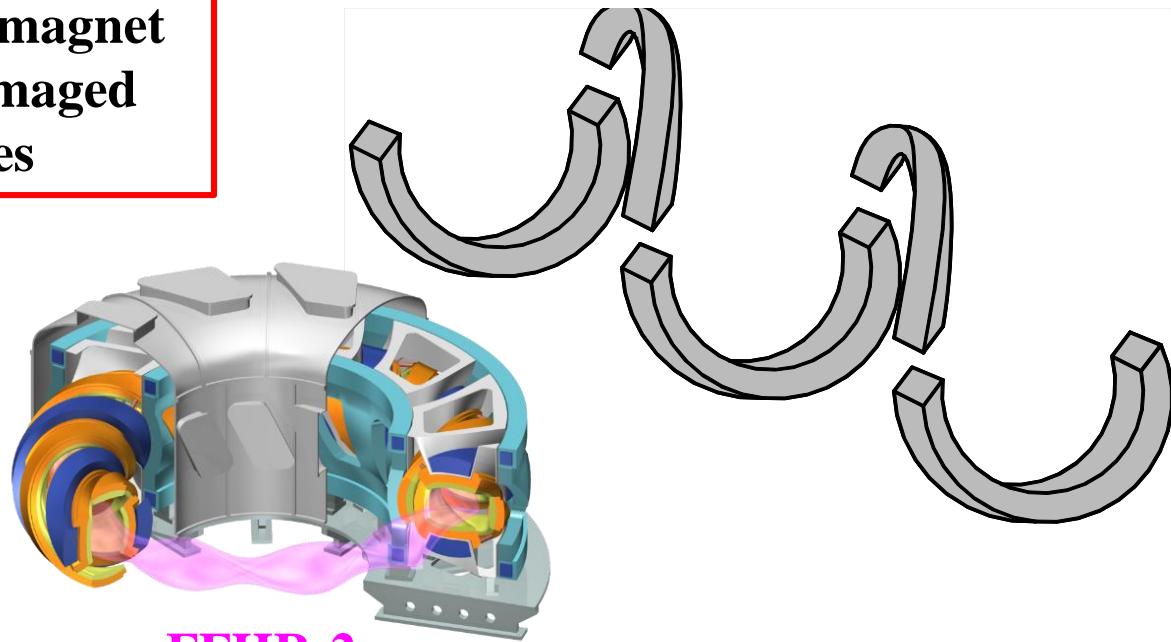
J. Plasma Fusion Res. SERIES 5 (2002) 532.

- (1) Construction cost reduction of magnet
- (2) Repair of magnet module if damaged
- (3) Maintenance of blanket modules



× 3~4

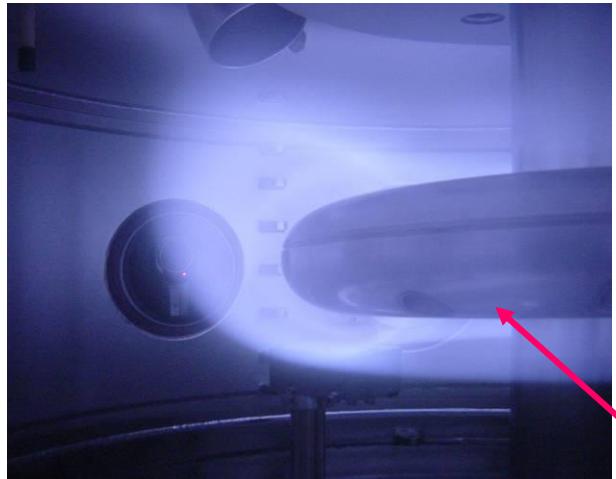
**LHD**  
continuous helical winding  
(1995-1996)



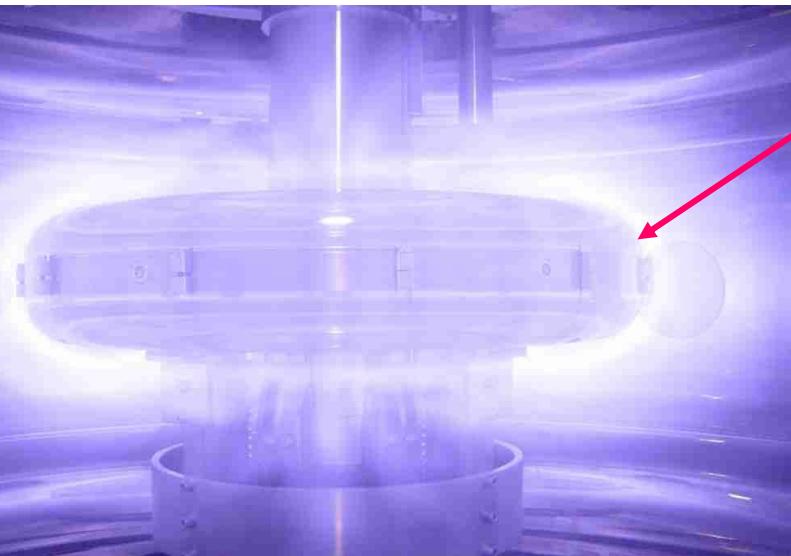
**FFHR-2**

# Application of HTS to Plasma Research

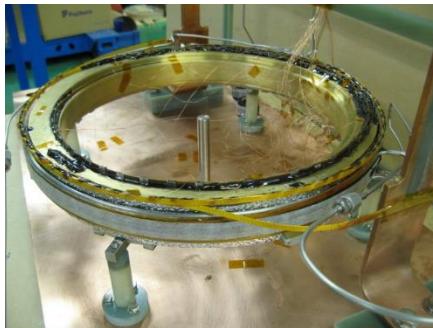
## RT Project at Univ. of Tokyo



Mini-RT (2003)



RT-1 (2006)



Upgrade to GdBCO (2012)

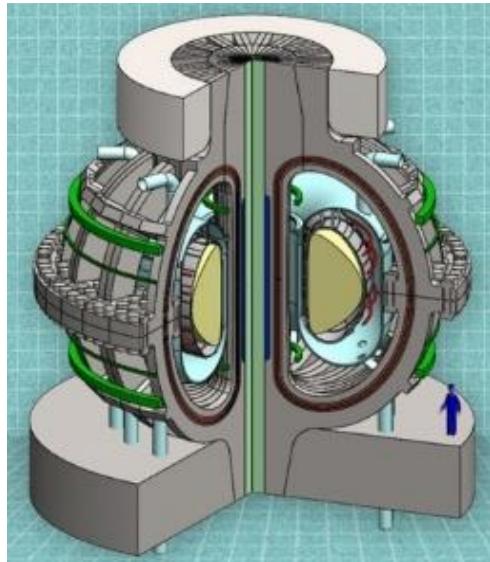


Bi-2223 HTS floating coils

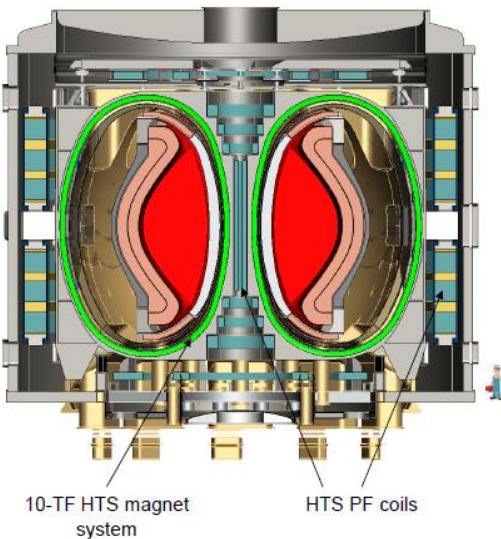


# HTS Magnet Concepts for Fusion in the World (2018)

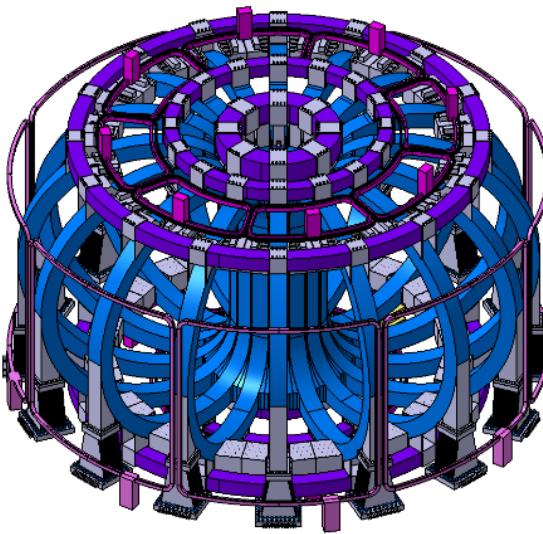
ARC (MIT, US)



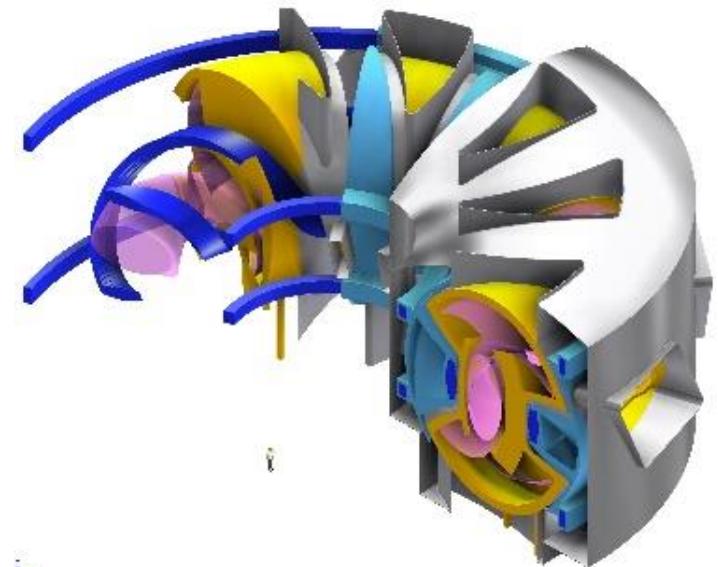
FNSF-ST (PPPL, US)



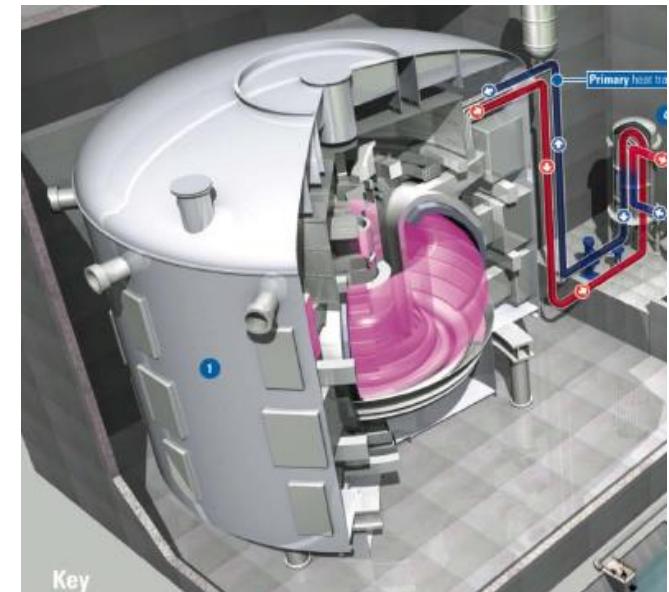
CFETR-Phase II  
(ASIPP, China)



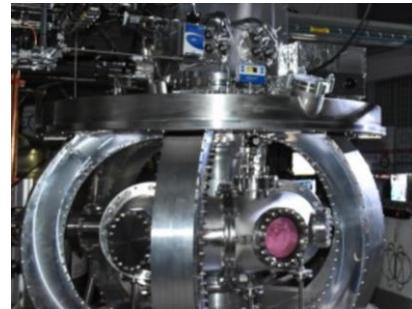
FFHR-d1 (NIFS, Japan)



EU DEMO HTS option (EUROfusion)

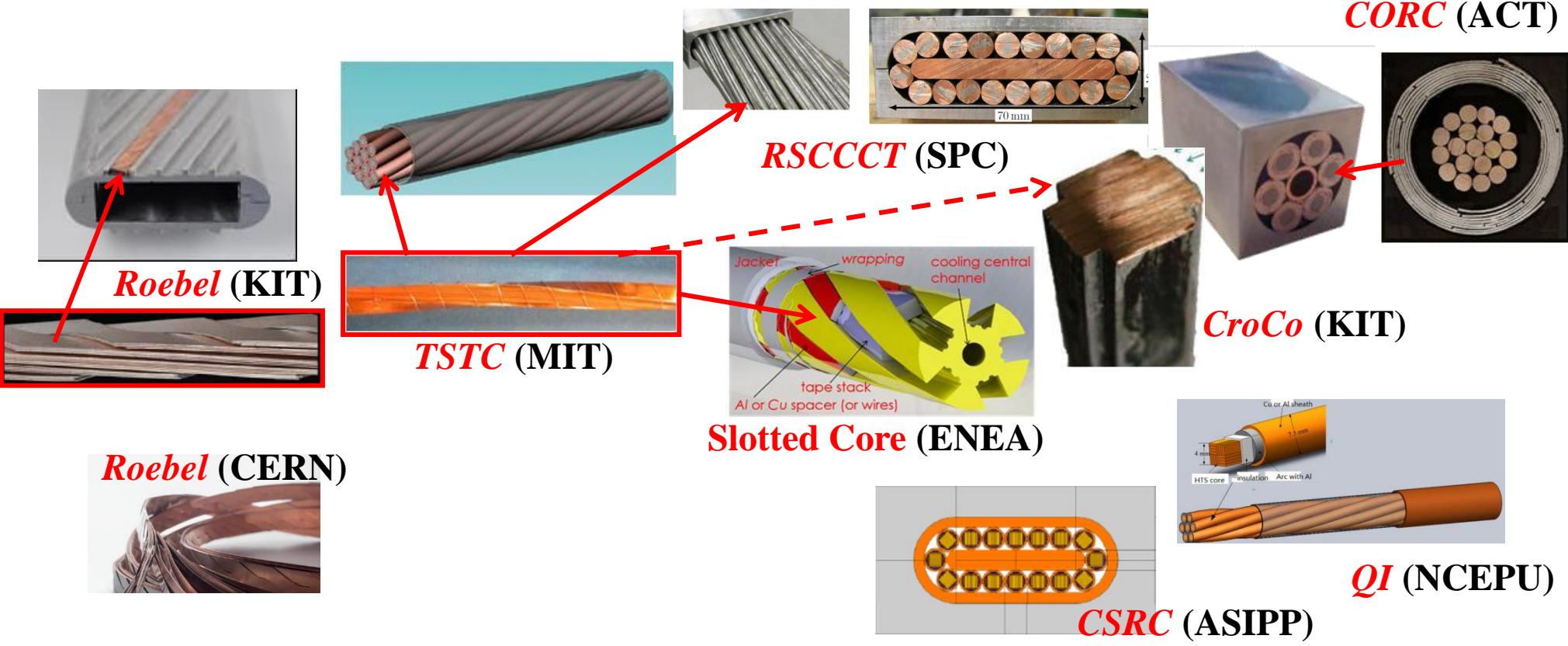


Tokamak Energy (UK)



# Large-Current HTS Conductors

## Twisted and Transposed REBCO Conductors

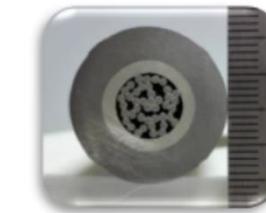


## Simply-Stacked REBCO Conductors



**STARS** (NIFS)

## Bi-2212 CIC Conductors

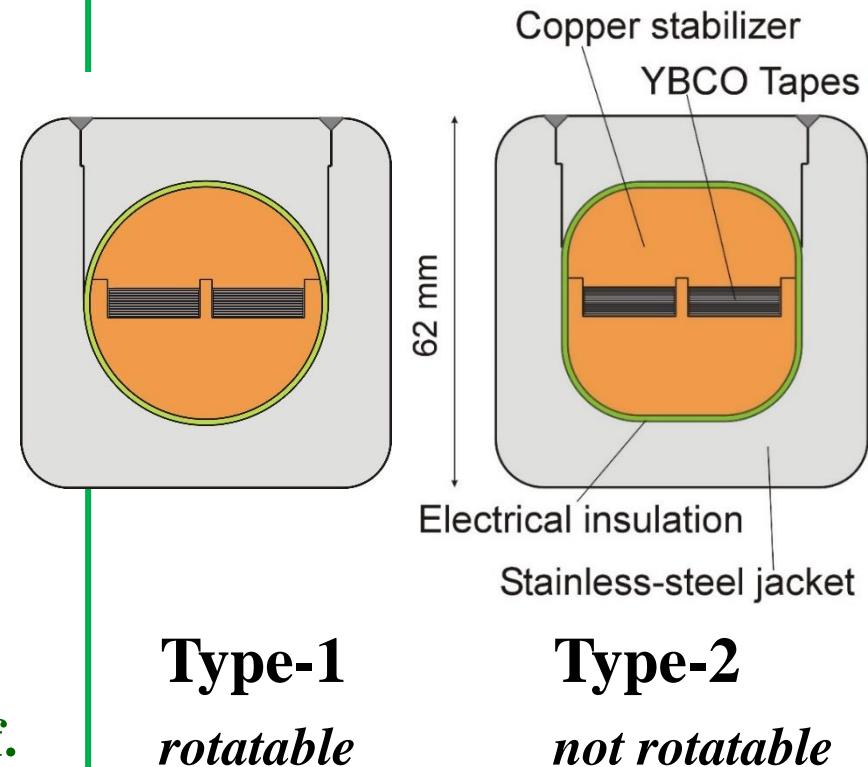


**Bi-2212** (ASIPP)

# 100 kA-class HTS Conductor for FFHR-d1

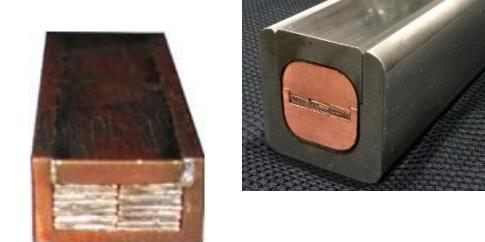
## "STARS" (Stacked Tapes Assembled in Rigid Structure)

Operation current	94 kA @ 12 T
Operation temperature	20 K
Conductor size	62 mm × 62 mm
Current density	24.5 A/mm <sup>2</sup>
Number of tapes	40
Cabling method	Simple Stacking
Stabilizer	OFC
Outer jacket	Stainless Steel
Electrical insulation	Organic or Inorganic
Cooling method	GHe or LNe
Superconductor	REBCO
Critical current	>900 A/cm @ 77 K, s.f.

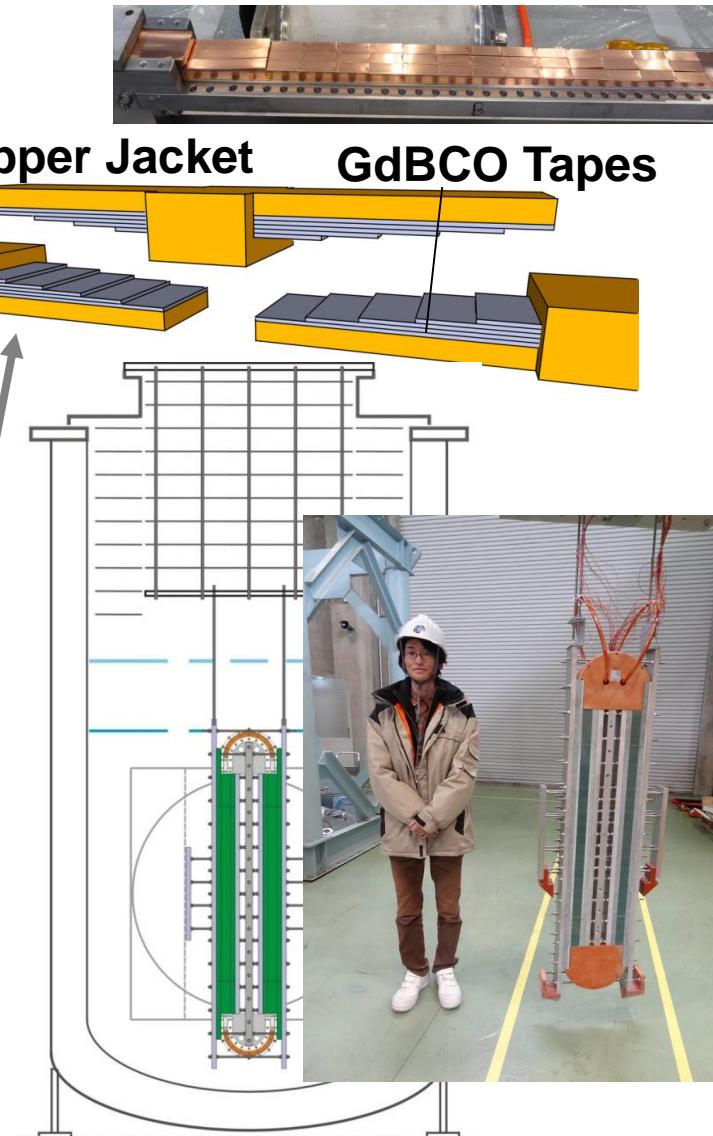
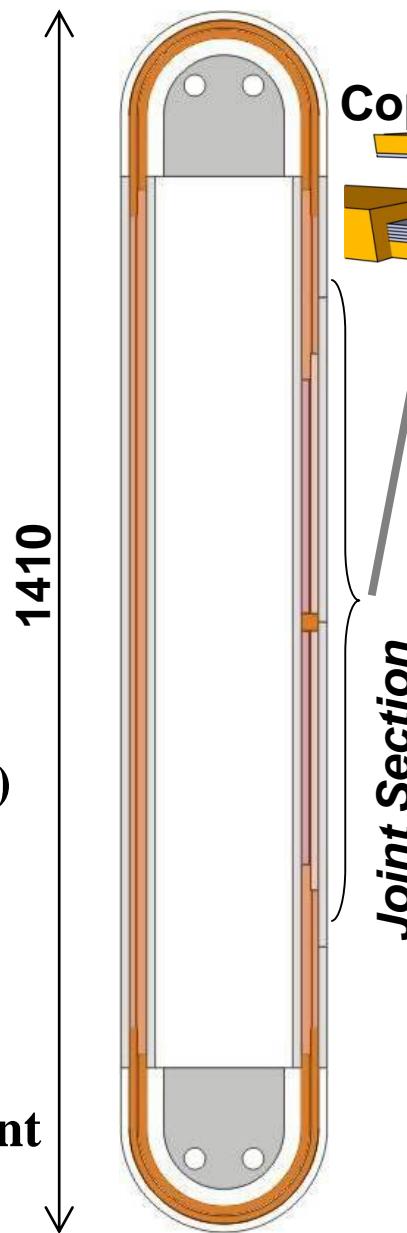
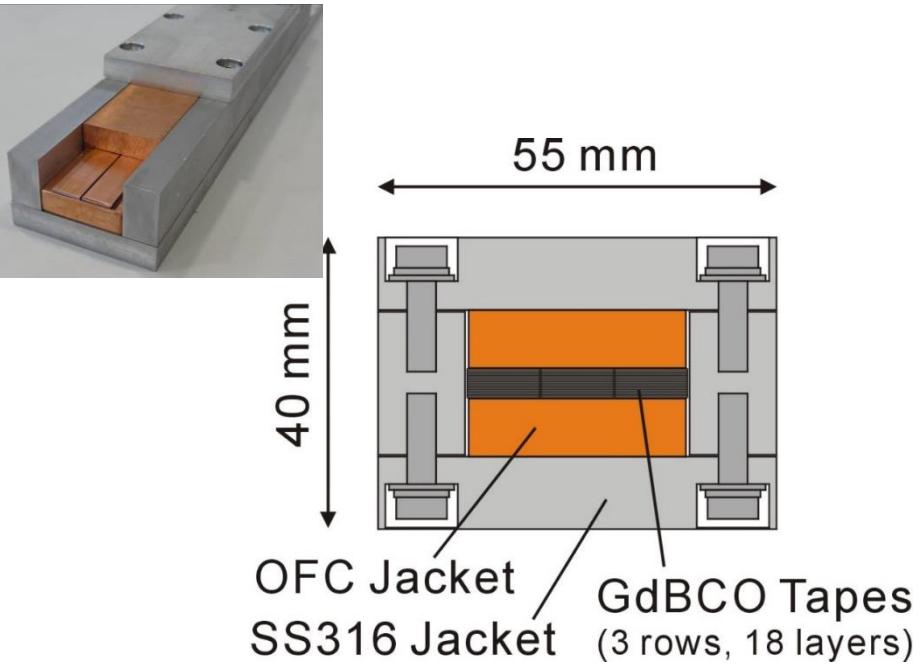


### Simply-stacked HTS conductor for DC helical coils

- Non-uniform current distribution may be allowed
- High mechanical strength (no void & no local deformation)
- Low cost and low resistance joint

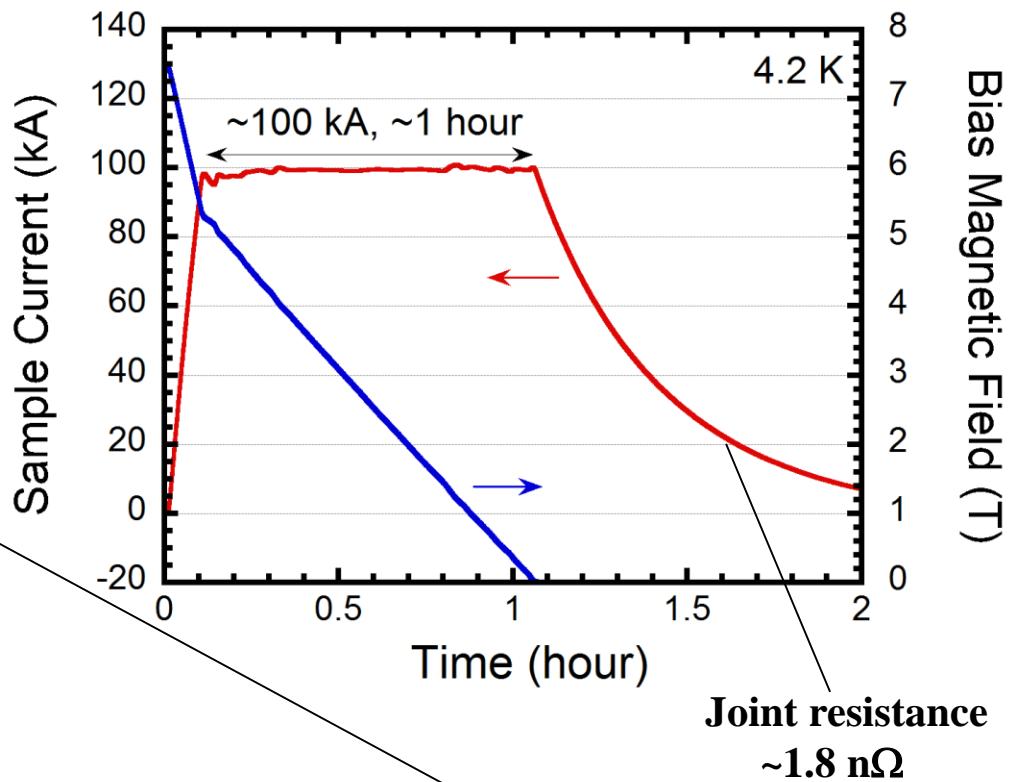
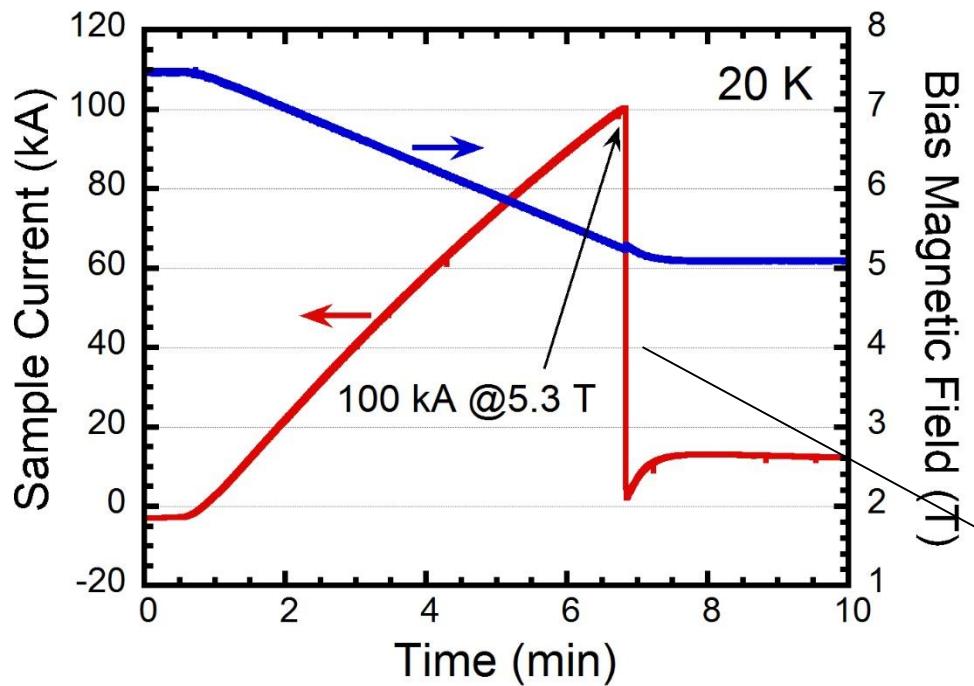


# 100 kA-Class Prototype Conductor Test



$$I_{sample} = \frac{M}{L_{sample}} I_{coil}$$

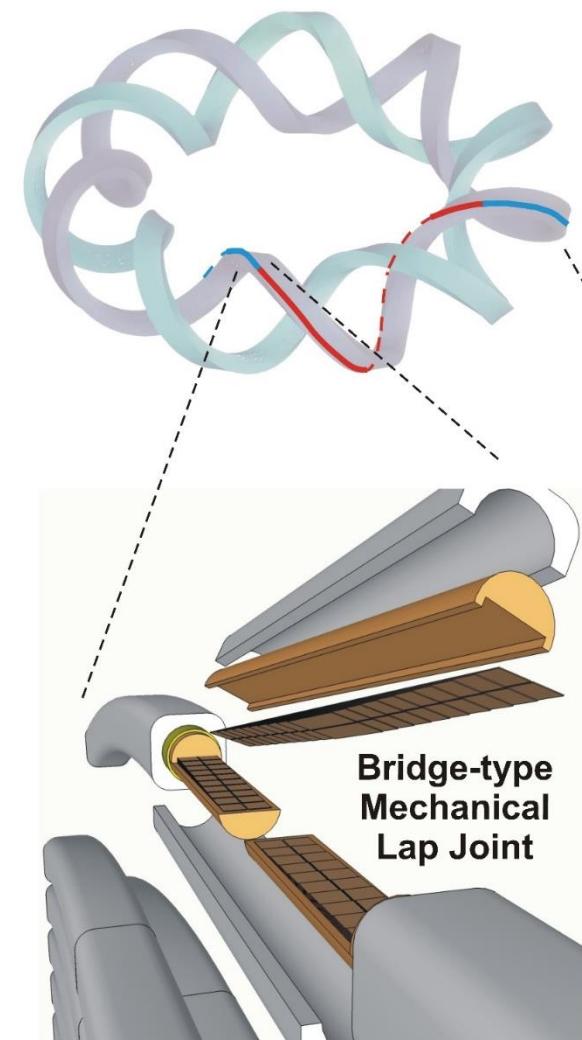
# 100 kA-Class Prototype Conductor Test



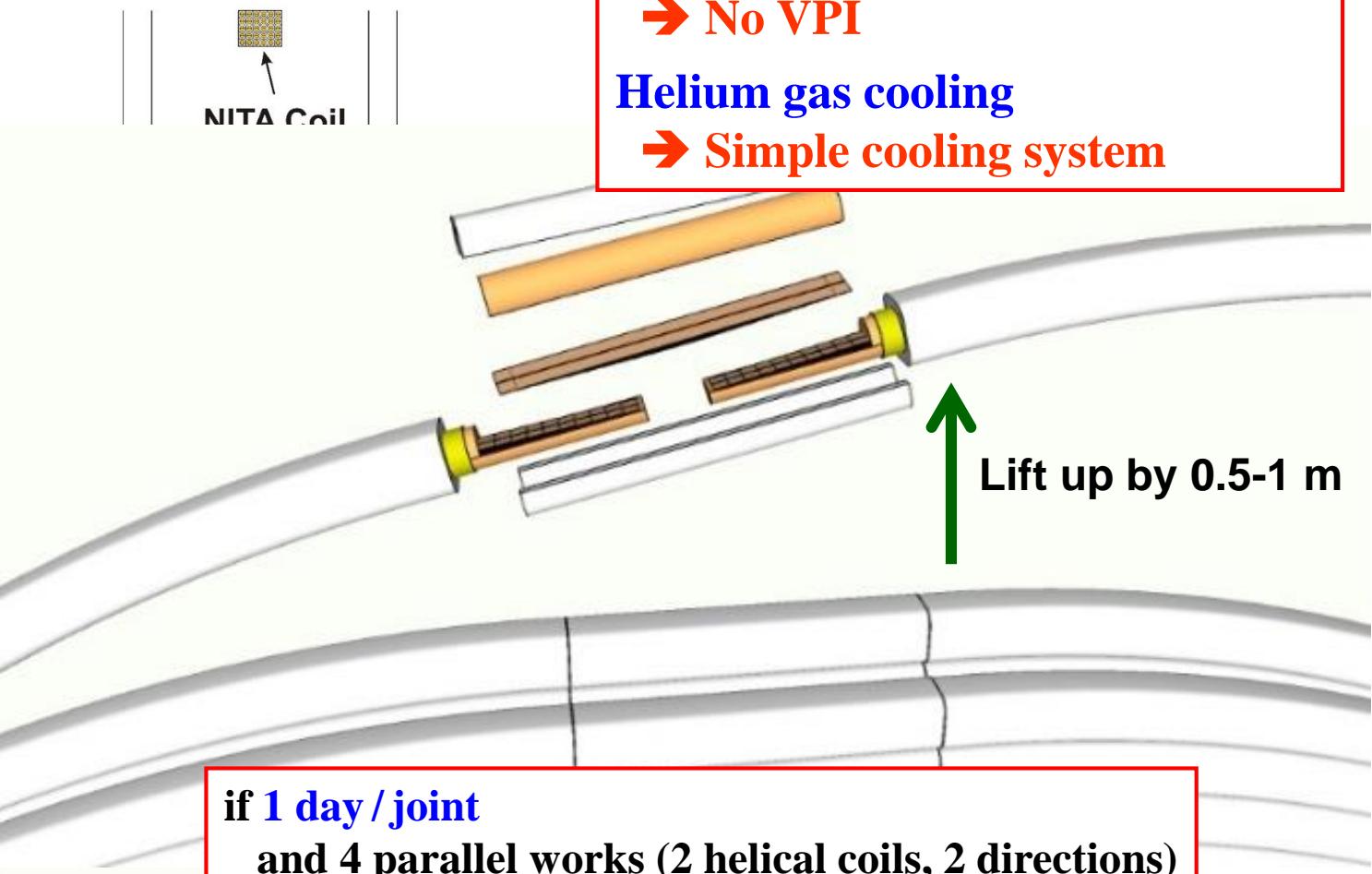
- 100 kA achieved @ 20 K, 5.3 T (quench)
- 118 kA achieved @ 4.2 K, 0.45 T (no quench)
- 100 kA current was successfully sustained for 1 hour @ 4.2 K
- Decay time constant : ~ 1000 s → Joint resistance : ~1.8 nΩ
- Quench occurred due to a failure in the joint manufacturing

# “Joint-Winding” of Helical Coils

## Helical Coils



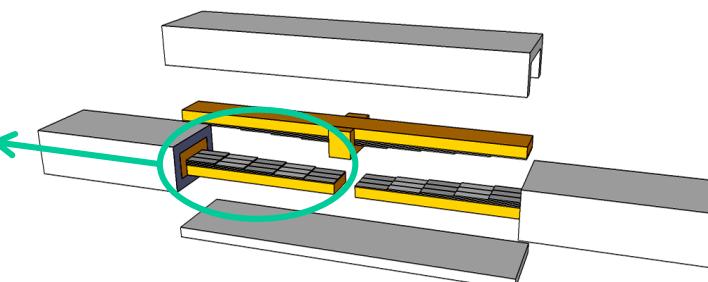
390 turns × 5 segments × 2 coils  
→ 3,900 joints



if 1 day / joint  
and 4 parallel works (2 helical coils, 2 directions)  
→  $3,900 / 4 = 2.7$  years

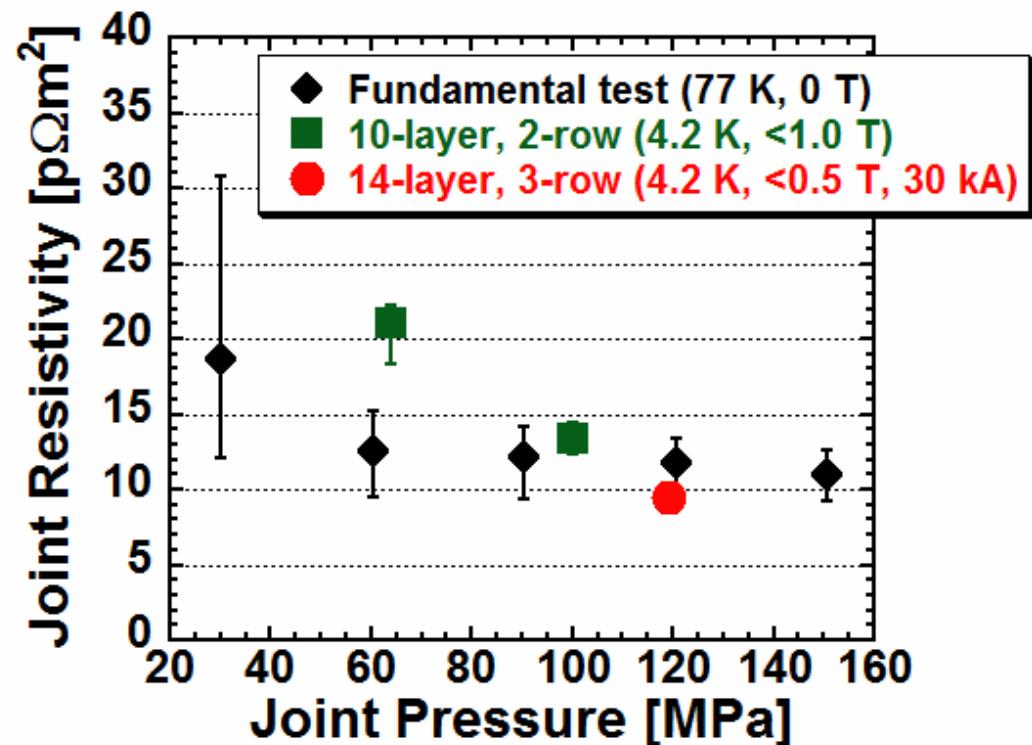
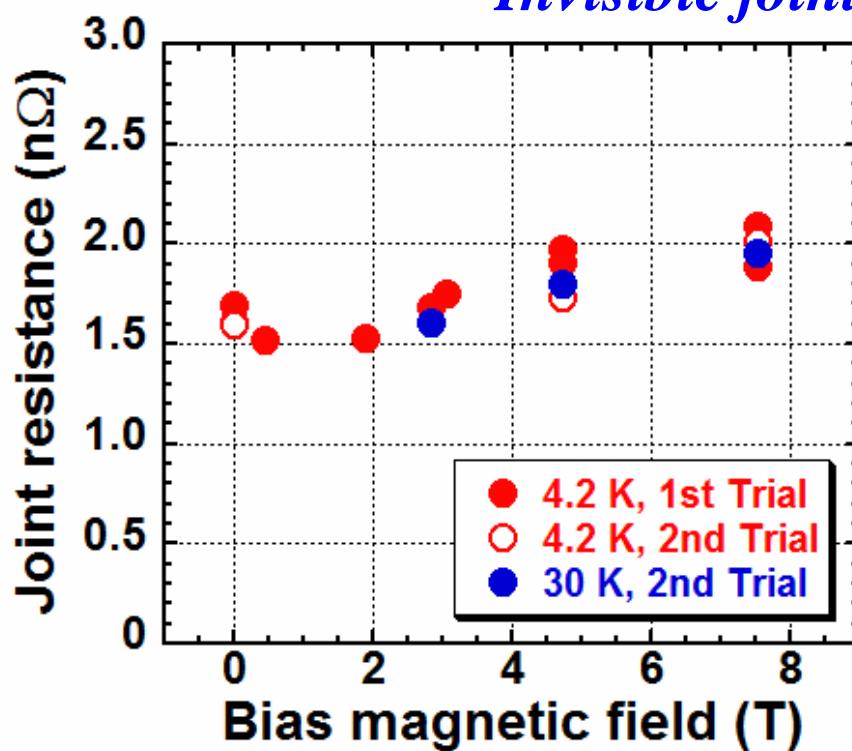
if 0.5 day / joint  
and 4 parallel works (2 helical coils, 2 directions)  
→  $3,900 / 2 / 4 = 1.3$  years

# Evaluation of Joint Resistance



Bridge-type mechanical lap joint  
*“Invisible joint”*

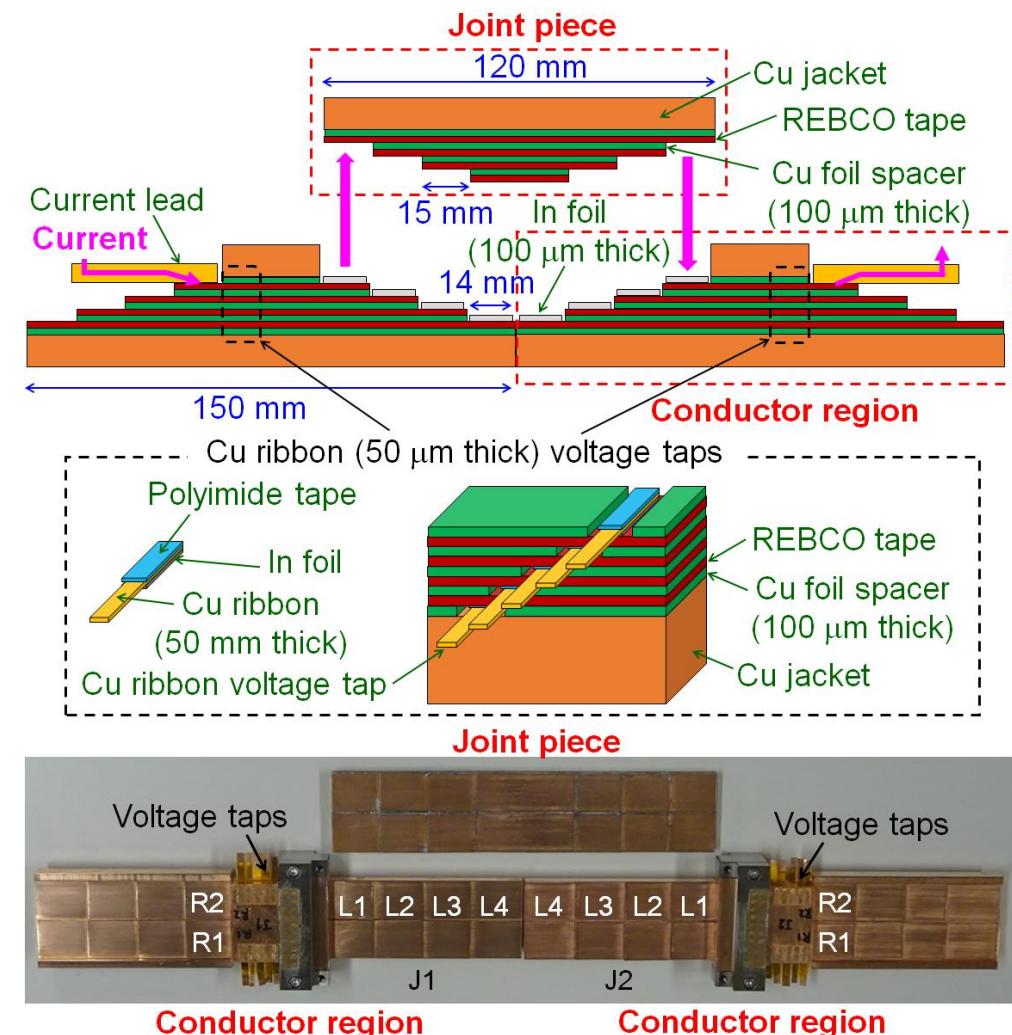
S. Ito (Tohoku Univ.)



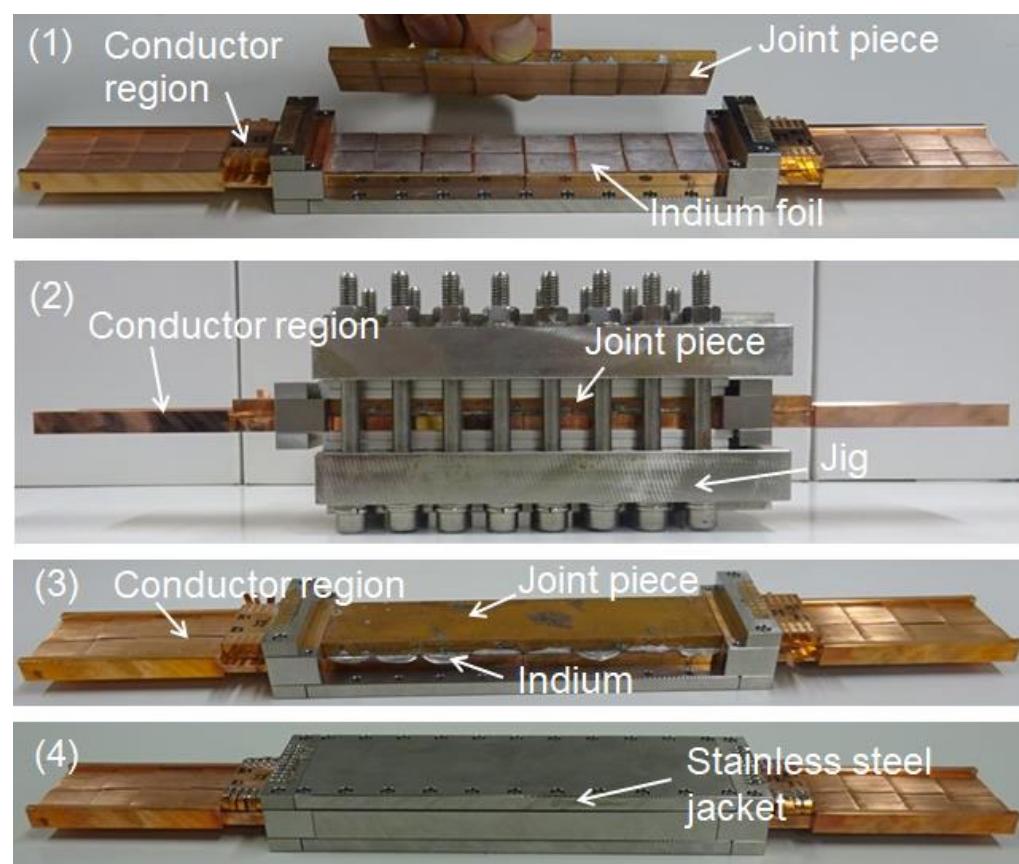
Joint resistance : ~2 nΩ → Joint resistivity : ~10 pΩm<sup>2</sup>

Required electrical power of the cryoplant at R.T. < 5 MW (for 3,900 joints)

# 一体式ジョイントピースを用いた 低抵抗機械的ラップ接続に成功



S. Ito, et al.  
presented in SOFT 2018  
to be published in FED



# LHD 後継計画 & NIFS 次期計画

LHD project 1998-2022

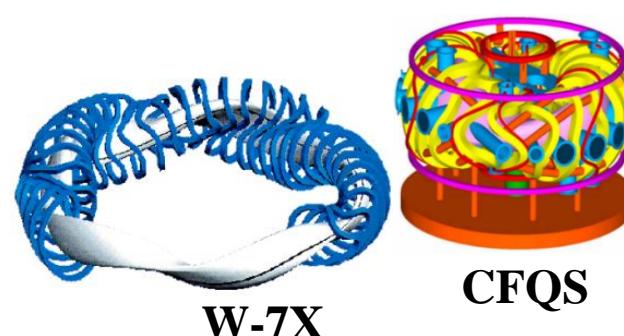
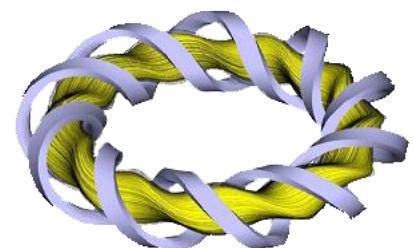
LHD upgrade 2023-2028

Divertor tiles will be changed from graphite to tungsten  
High-power steady-state plasma production (2 MW x 3 hours)

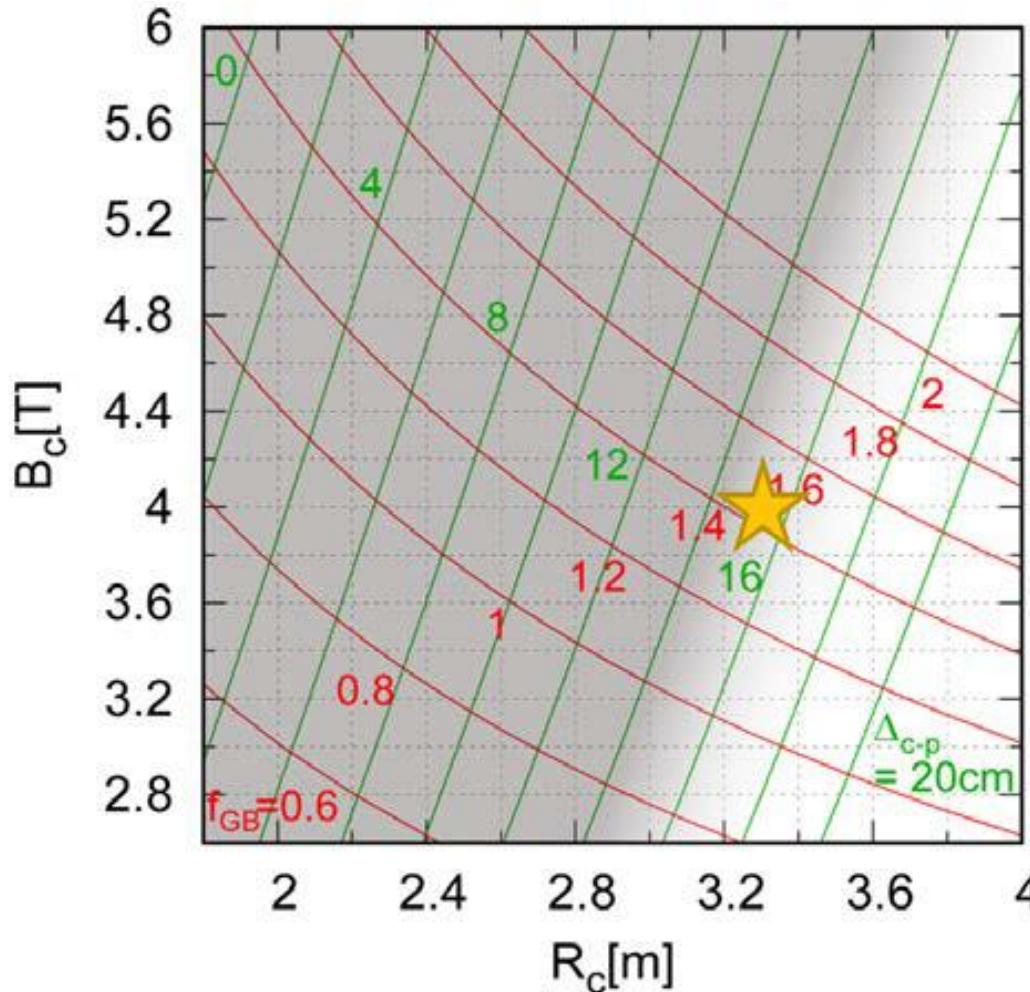
Post-LHD project 2029-2038

Presently, two proposals are being examined in parallel

- HTS heliotron by optimizing LHD magnetic configuration
- Modular stellarator with new magnetic configuration



# HTS ヘリカル装置の大きさと磁場強度



T. Goto, K. Takahata

プラズマと第一壁との距離を十分とるために、

ヘリカルコイル巻線部の電流密度: 80 A/mm<sup>2</sup>

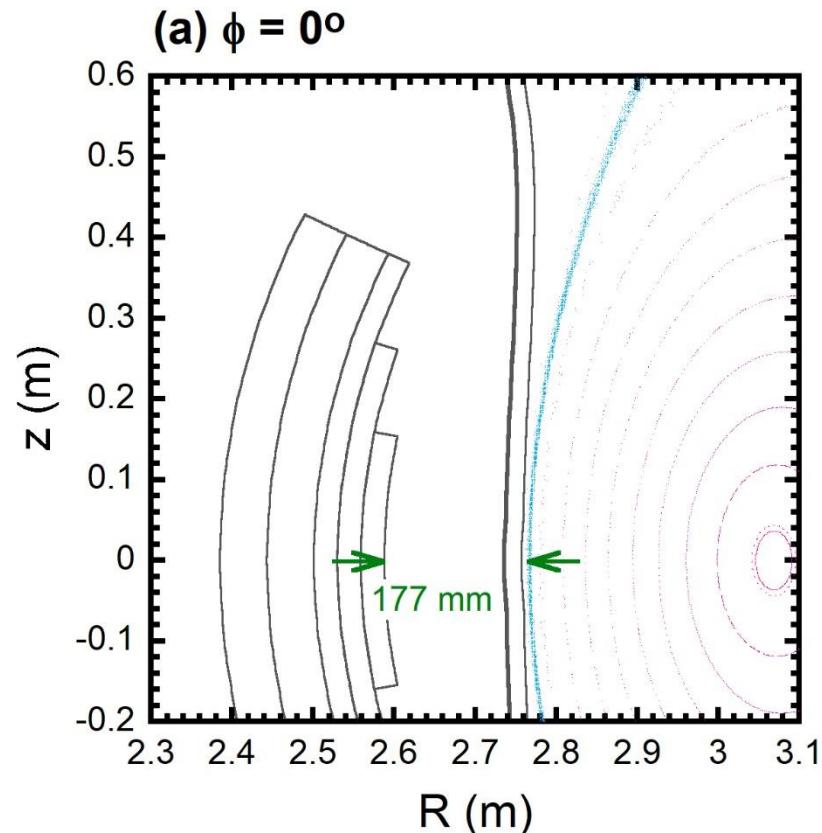
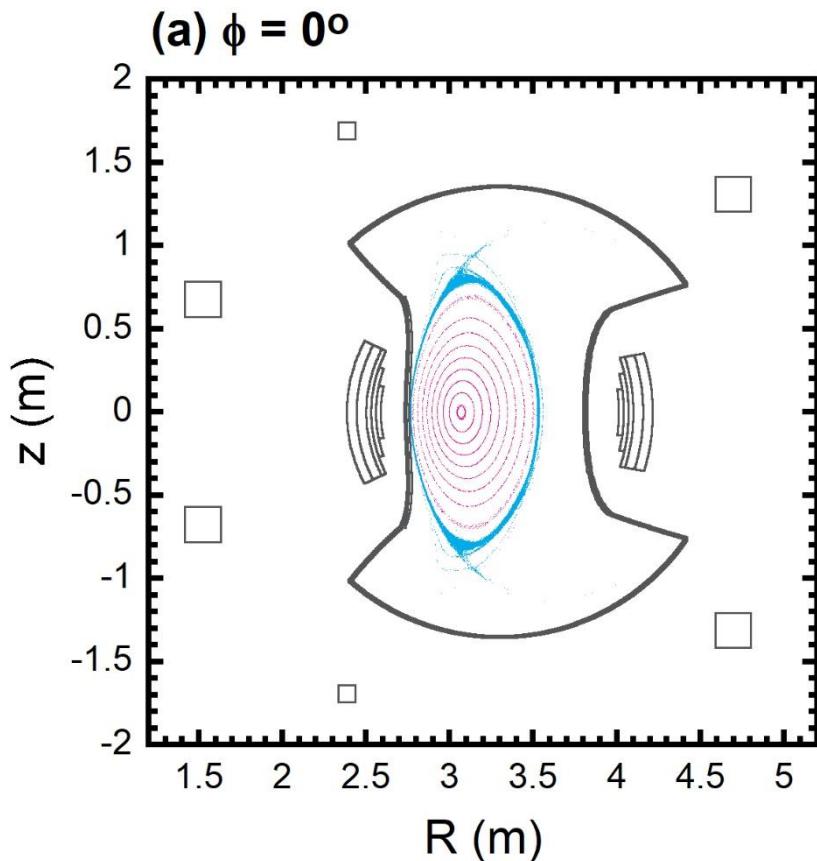
→  $R = 3.3$  m,  $B = 4$  T  
(現在の候補)

LHD :  
 $R = 3.9$  m,  $B \sim 2.7$  T  
40 A/mm<sup>2</sup> (設計)  
(35 A/mm<sup>2</sup> 実績)

大型コイルにかかわらず、高い電流密度が要求される

→ クエンチ保護が課題 → 短時定数遮断(耐電圧)、早期検出、クエンチヒータ、無絶縁？

# Vacuum Magnetic Surfaces of a LHD-similar configuration with $j_{HC} = 80 \text{ A/mm}^2$

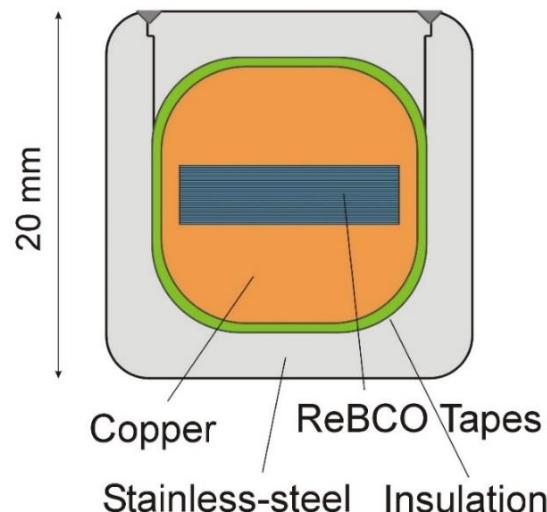


トーラス内周部におけるエルゴディック層境界とヘリカルコイル巻線部最内層との距離:  $177 \text{ mm} > 150 \text{ mm}$

# Three Candidate Conductors (HTS 10 kA-class)

## STARS

(Stacked-Tapes Assembled in Rigid Structure)

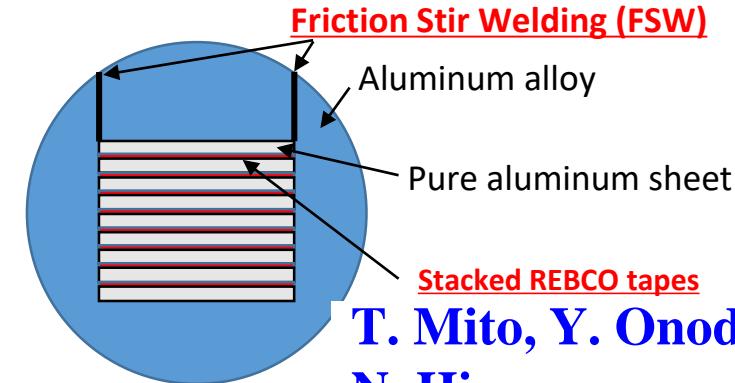


N. Yanagi, Y. Terazaki, S. Matsunaga,  
S. Ito (Tohoku Univ.)

J. Miyazawa,  
Y. Narushima,  
S. Matsunaga

## FAIR

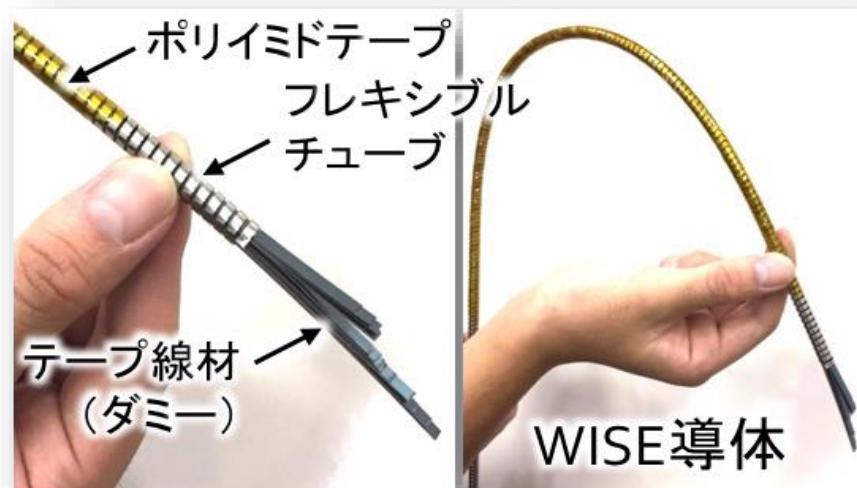
(FSW, Al-alloy, Indirect-cooling, REBCO)



T. Mito, Y. Onodera,  
N. Hirano

## WISE

(Wound and Impregnated Stacked Elastic tapes)



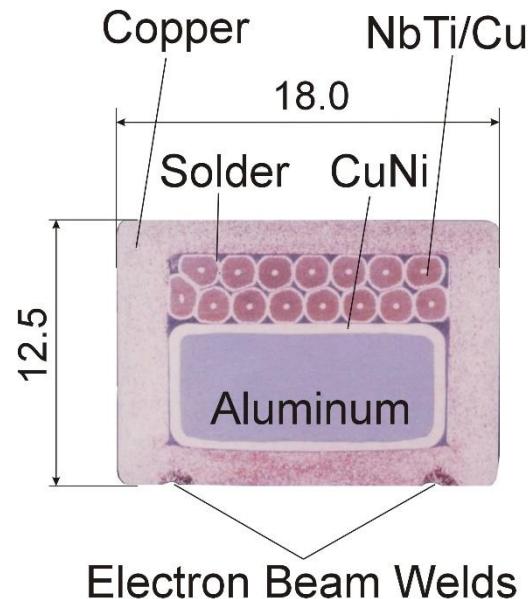
# Conductor for Helical Coils

## LTS vs. HTS

### LTS (for LHD)

NbTi/Cu + Al + Cu

13 kA @ 6.9 T, 57.8 A/mm<sup>2</sup>

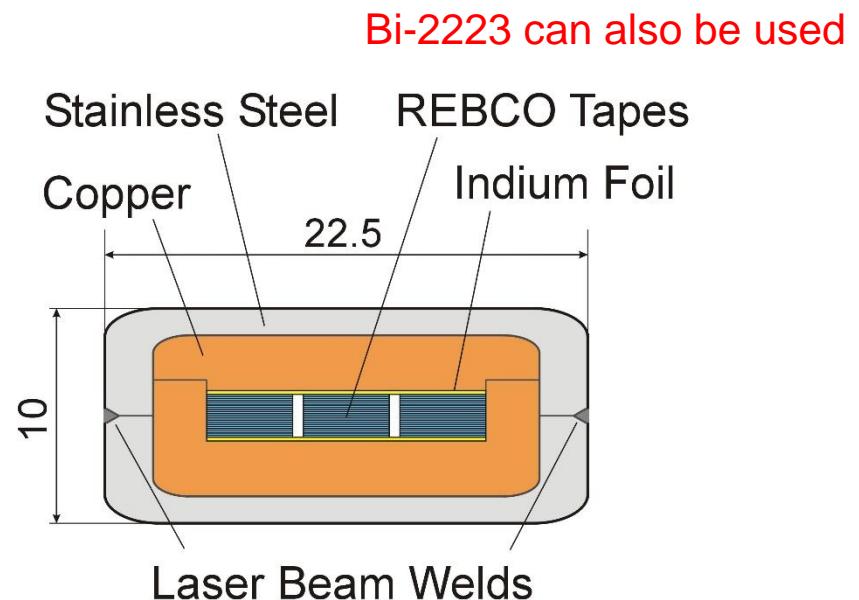


Effective Young's modulus: 100 GPa

### HTS (for Post-LHD)

REBCO + Cu + SS

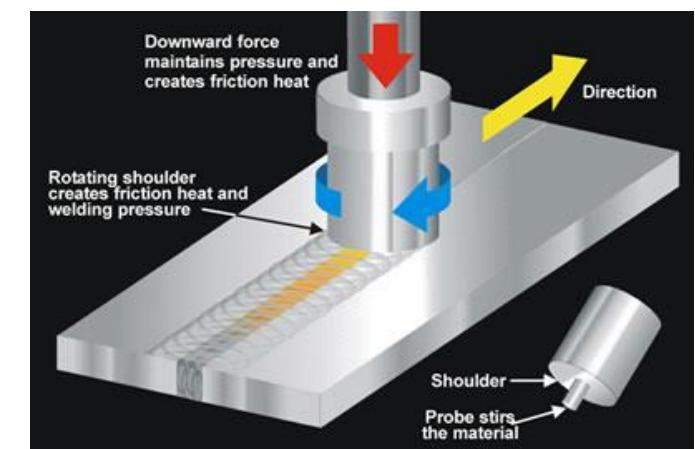
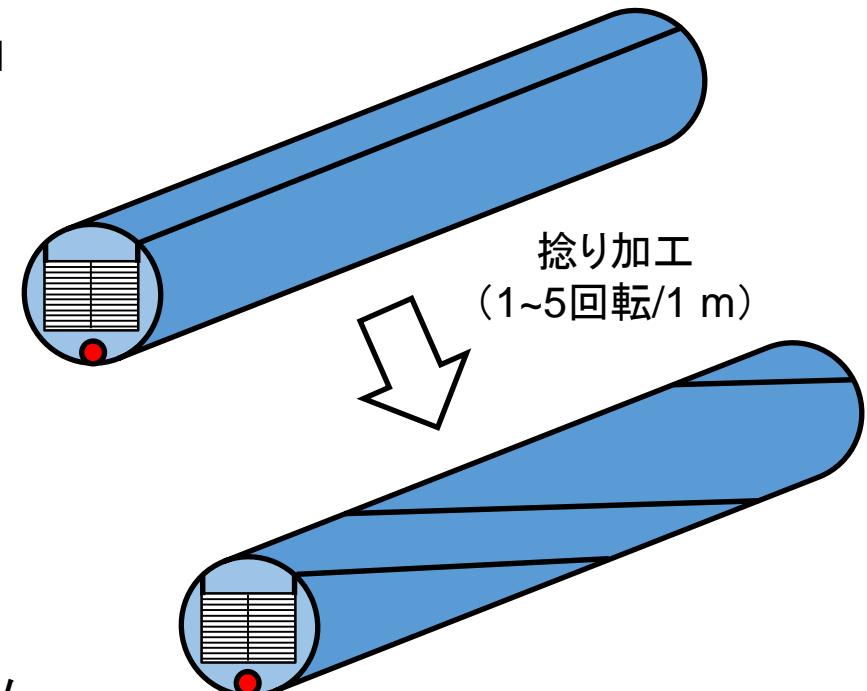
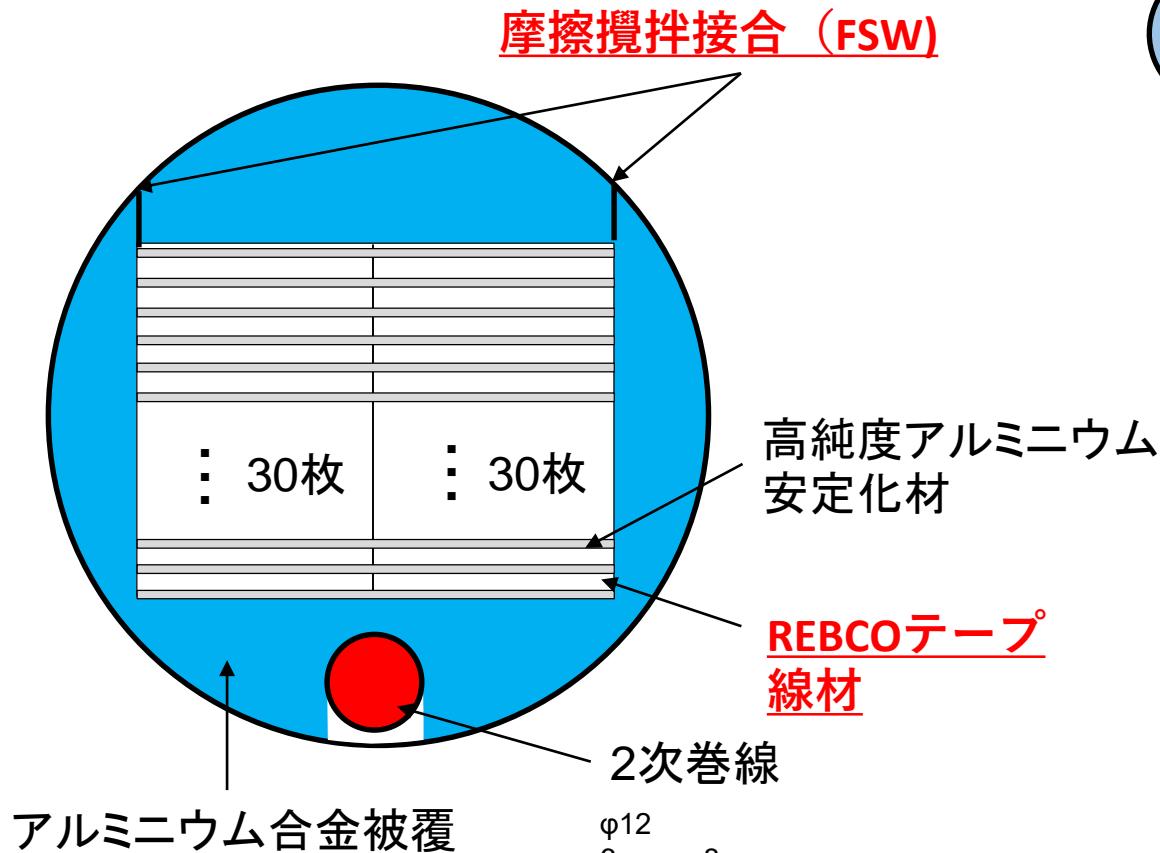
18 kA @ ~10 T, 80 A/mm<sup>2</sup>



Effective Young's modulus: 150 Gpa  
*Similar bending (winding) by further flattening*

Bi-2223 can also be used

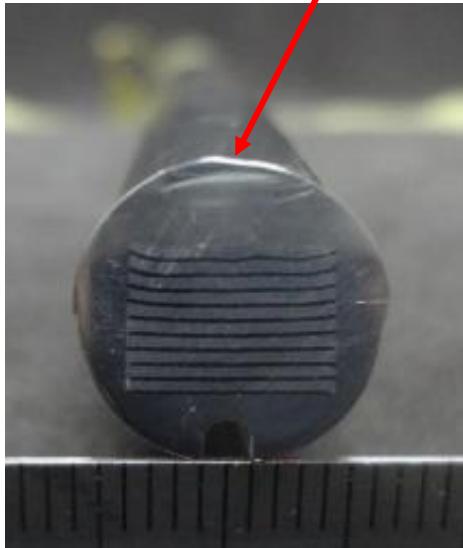
摩擦攪拌接合(FSW)、アルミニウム合金被覆(Aluminum alloy jacket)、間接冷却(Indirect cooling)、REBCO導体の頭文字をとってFAIR導体と命名



Friction stir welding (FSW)

# FIRST TRIAL PRODUCTION OF FAIR CONDUCTOR

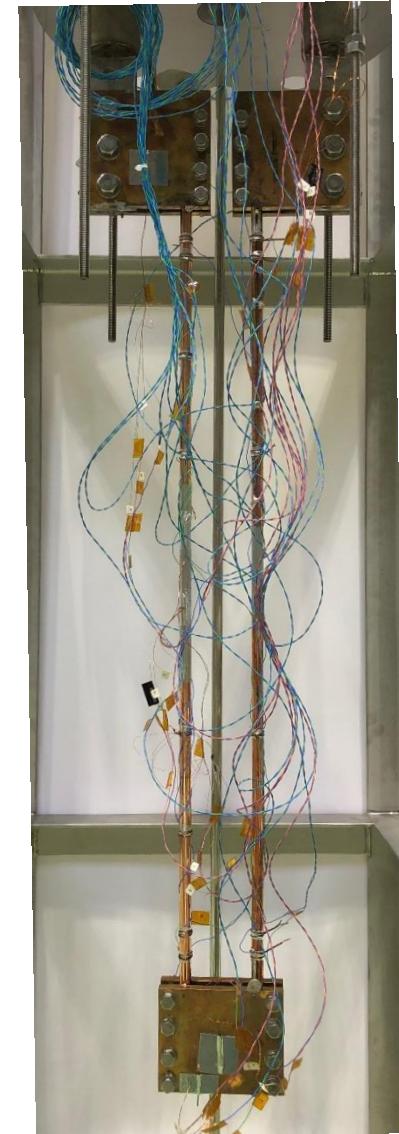
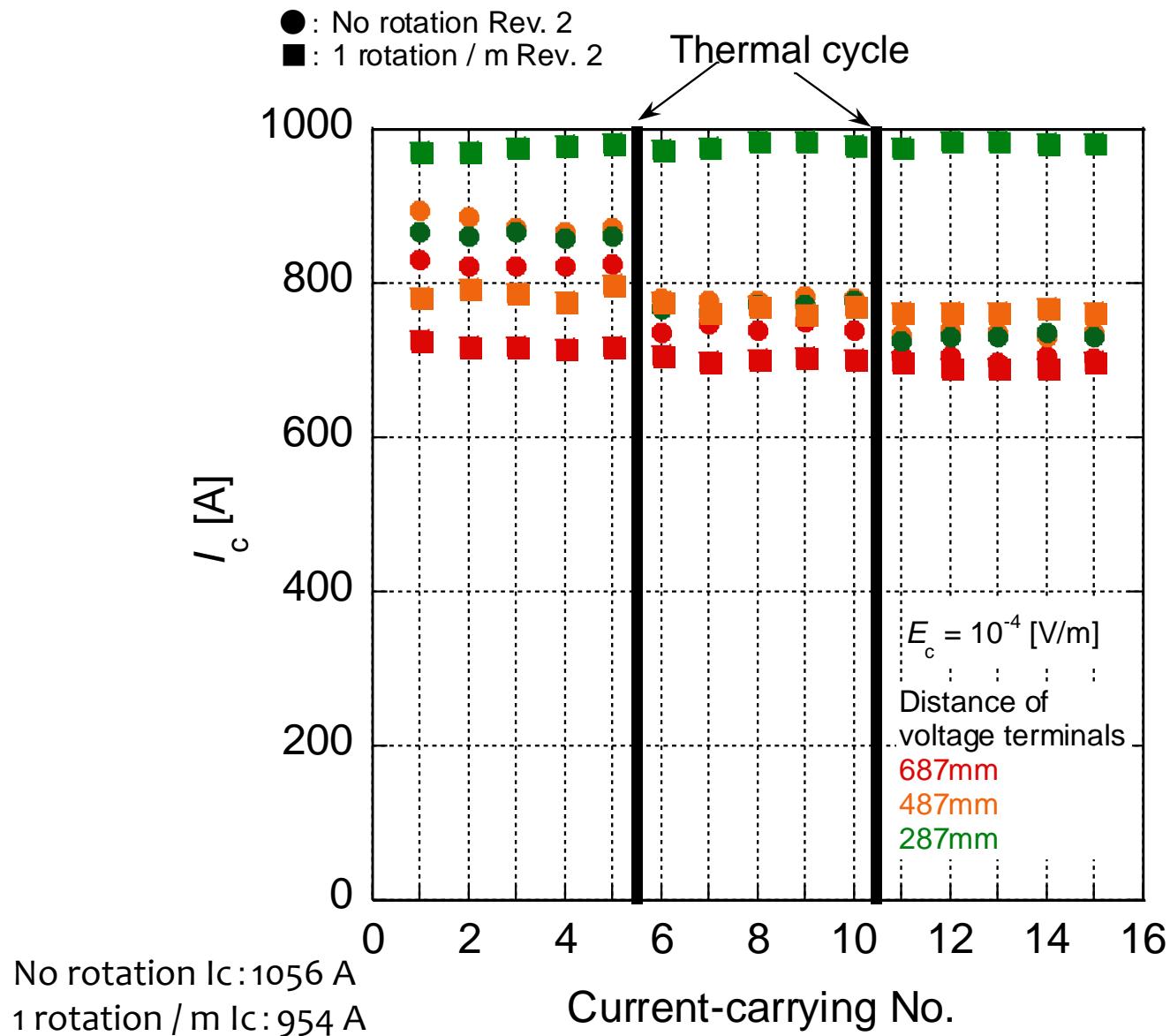
Cross section of FAIR conductor



Twist pitch: 2 rotations / m

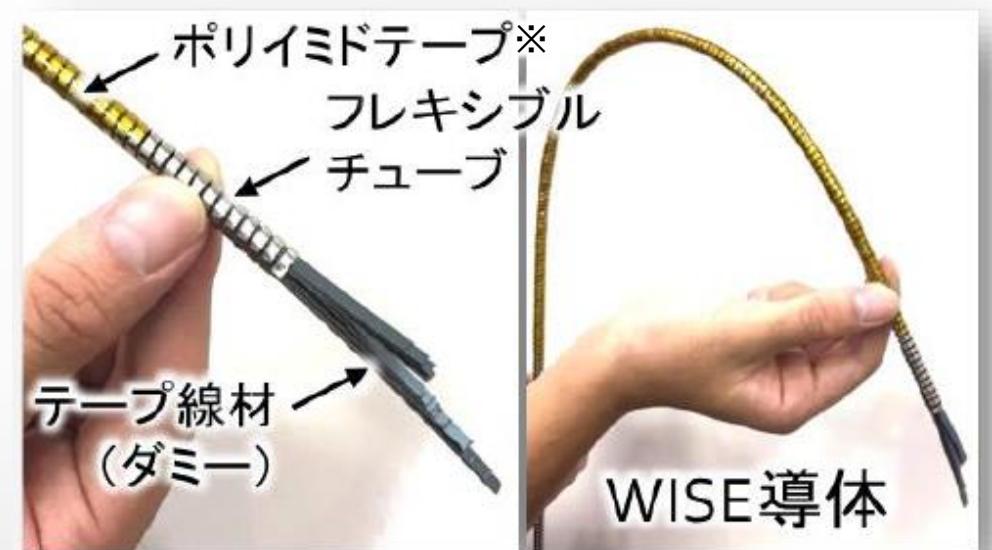
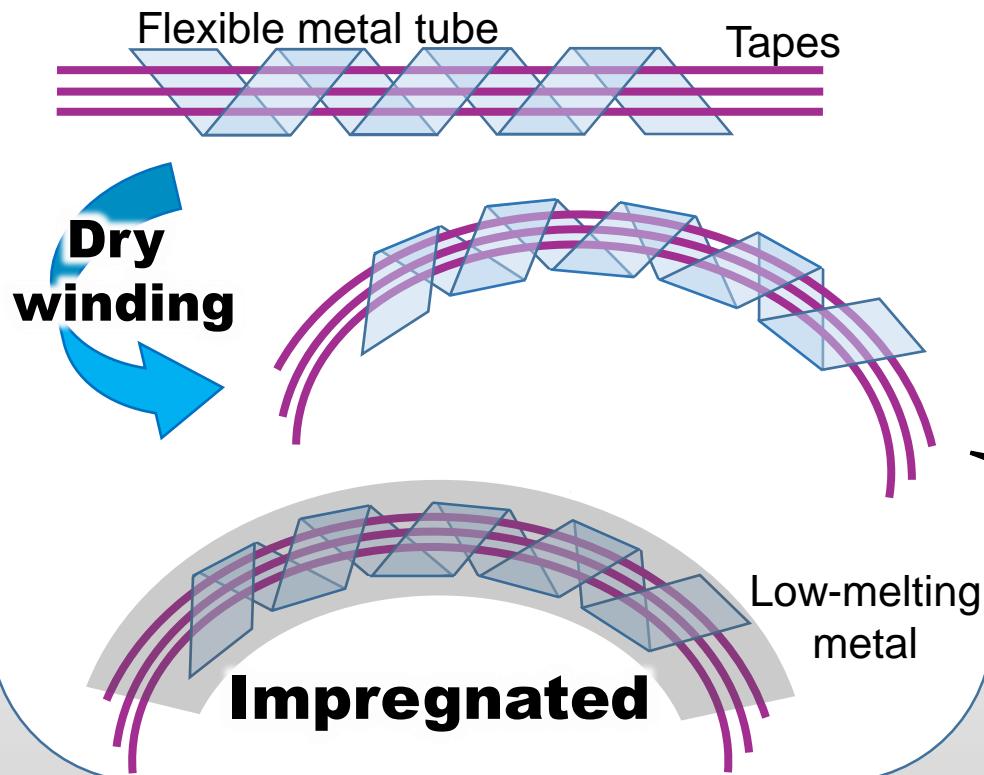
REBCO tapes : SuperPower Inc. SCS4050-AP

# TEST RESULTS OF THE SECOND TRIAL PRODUCTION CONDUCTOR



# “Wound and Impregnated” on the WISE concept

## Wound-and-Impregnated-Stacked Elastic tapes (WISE) concept



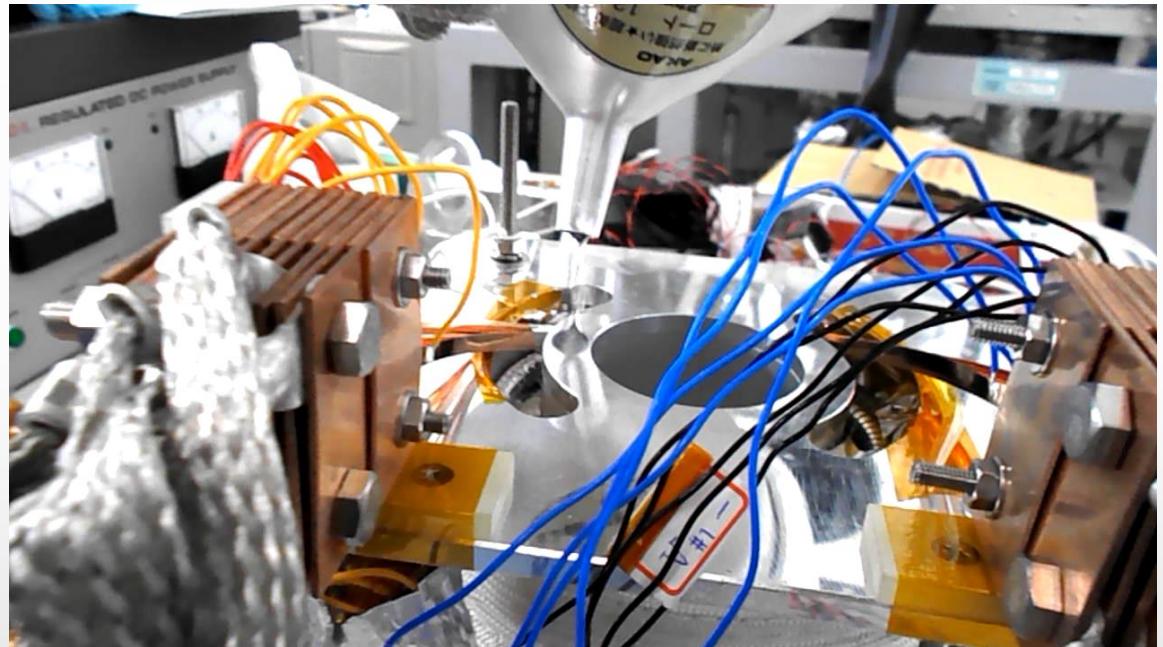
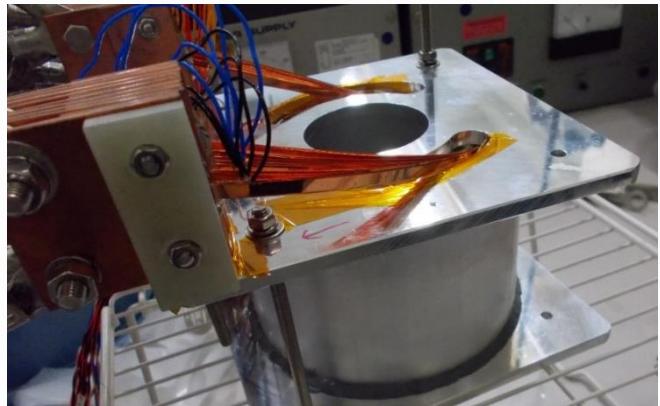
\*ポリイミドテープなしも可

Tapes naturally deform  
⇒ No hard bending

# Wound and Impregnated

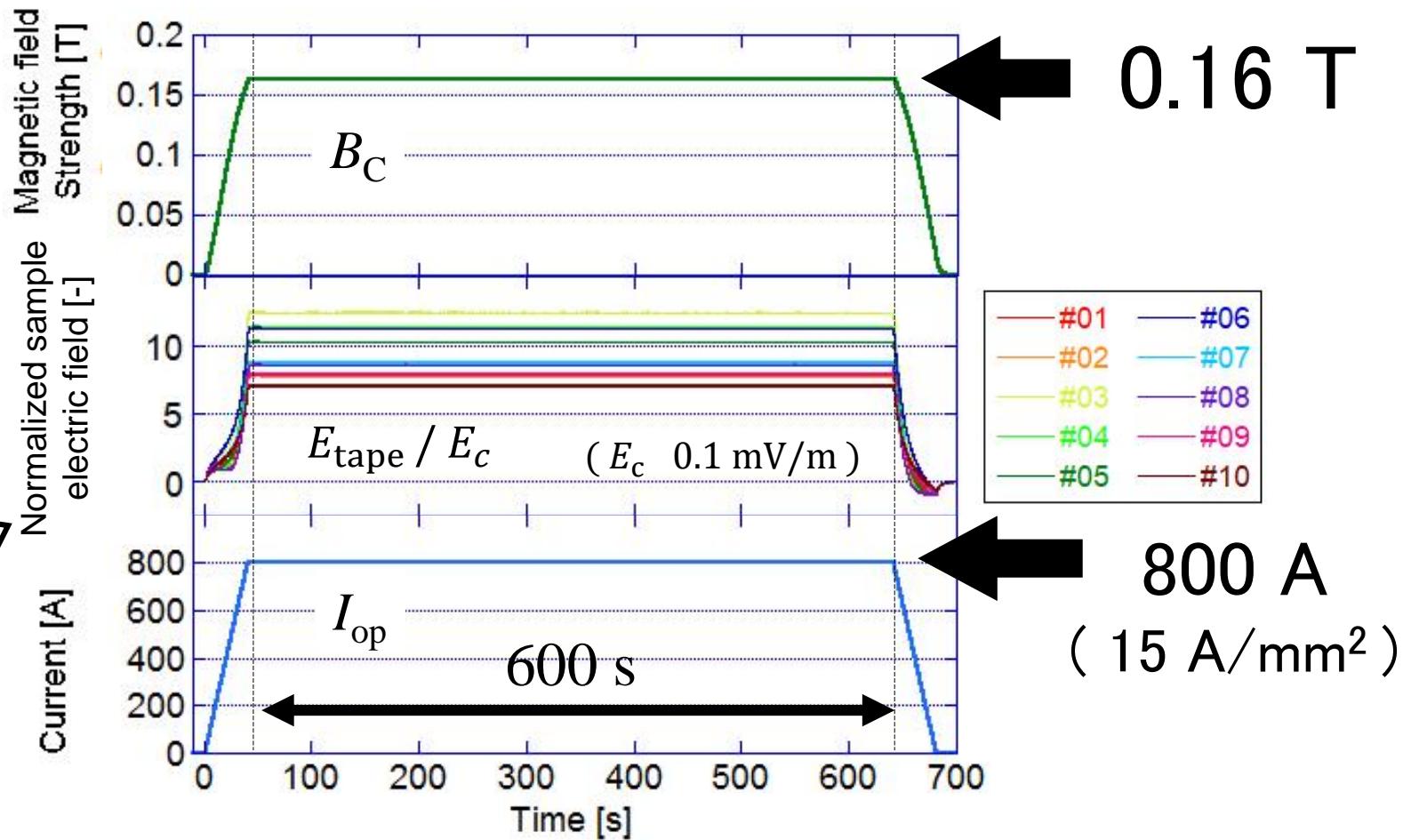


**Handmade winding**

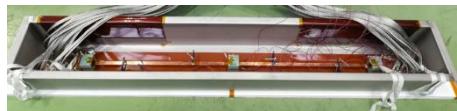


# Achieved 0.16 T @ 800 A, 77 K

Despite  $\frac{E_{\text{tape}}}{E_c} \sim 10$ ,  
quench did not  
occur.



# Development Plan of HTS-STARs Conductor for the Next-Generation Helical Device

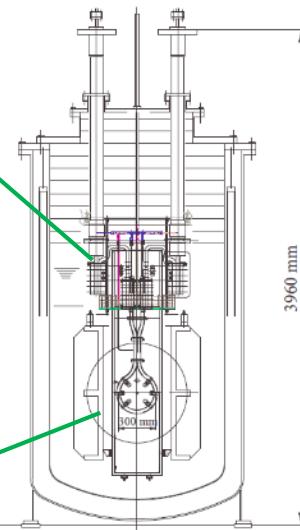


Short sample (~3 m) test  
Straight @77 K, 0 T



Short sample (~3 m) test  
Bent radius :1-0.2 m  
@77 K, 0 T

Short sample (~3 m) test  
Coiled ( $\phi 0.5$  m) @20 K, 9 T



FY2019

FY2020

FY2021

Short length (~3 m)

Longer length (> 3 m)

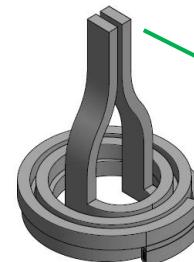
Conductor test

Magnetic field 0 T

9 T

13 T

Long conductor (~10 m)

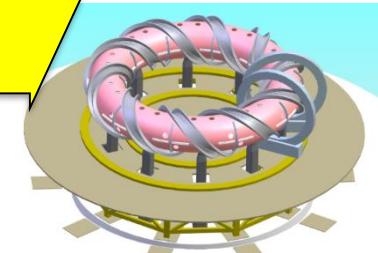


Short sample (>3 m) test  
Coiled ( $\phi 0.6$  m) @20 K, 13 T



Mechanical lap-joint  
(developed by Tohoku Univ.)

3D helical coil winding  
technology



# Helical Coil Winding

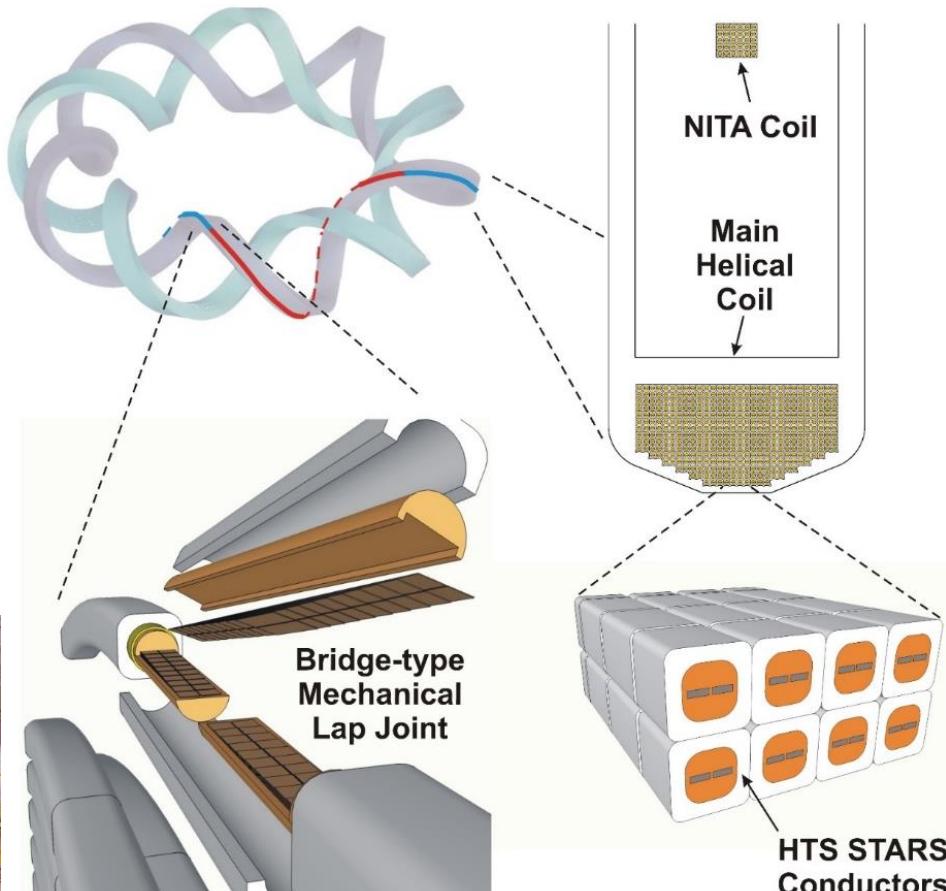
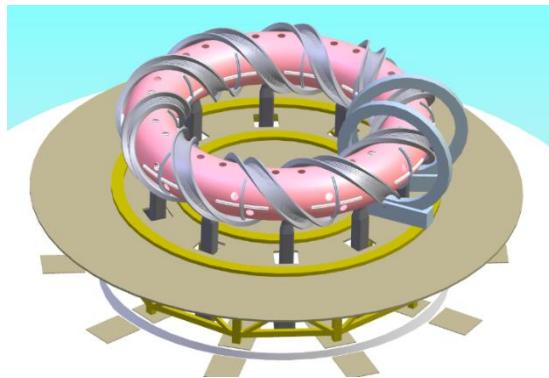
## *Continuous vs. Joint*

### ● Continuous Winding

- Experience by LHD construction
- Long conductor (~ 1 km) necessary
- Optimized twisting angle

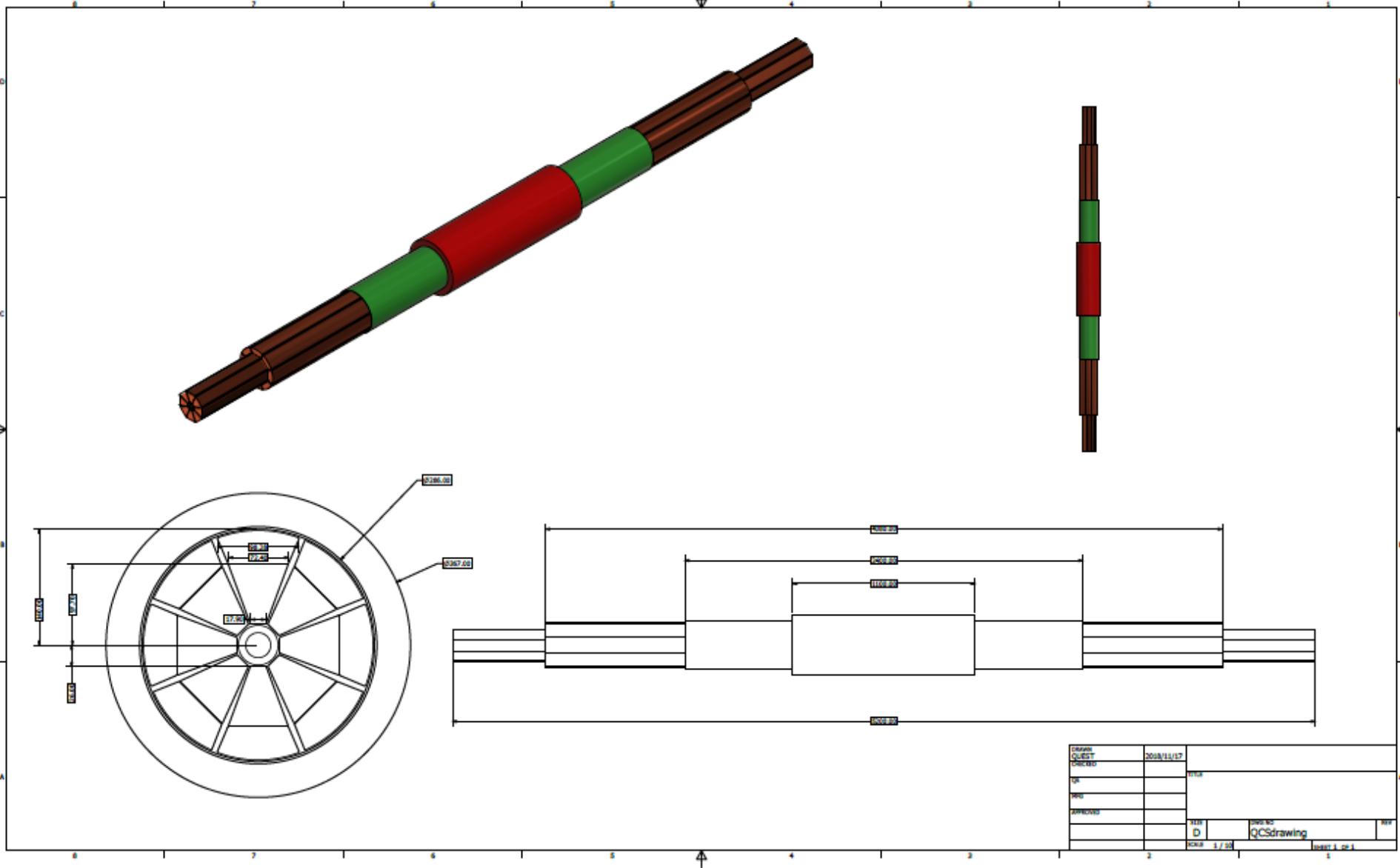
### ● Joint-Winding

- Challenging but rewarding
- ~ 4000 joints = high risk
- Industrial robot (manufacturing and inspection)



# NIFS 次期計画への適用をめざした大電流 HTS 導体の開発

- 高電流密度 HTS 導体
  - 80 A/mm<sup>2</sup> 目標
- 3種類の候補導体を並行して開発に着手
  - STARS, FAIR, WISE
- クエンチ保護
  - 短時定数遮断、早期検出
  - 2次巻線によるクエンチバック
  - 無絶縁巻線の可能性(励磁時定数、コイル全体温度上昇等、課題)
- ヘリカルコイル巻線
  - 最適な連続巻線方法の考案
  - ヘリカルコイルの傾き角調整によるエッジワイズ曲げ歪の低減
  - 接続巻線を早期に確立(産業用ロボットによる施工)
- 今後の開発計画
  - 短尺試験、長尺試験 → 3年以内ターゲット
  - コイル試験、ヘリカル巻線 → 5年以内ターゲット

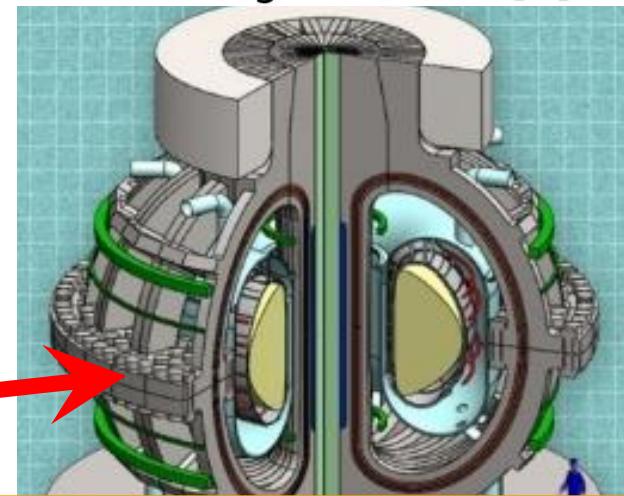
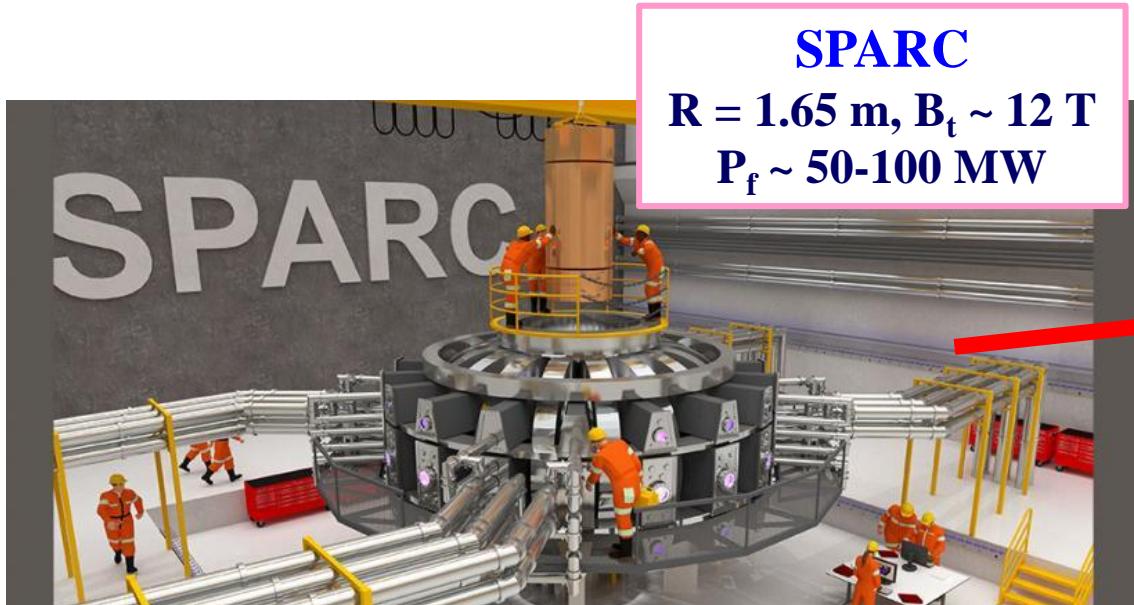
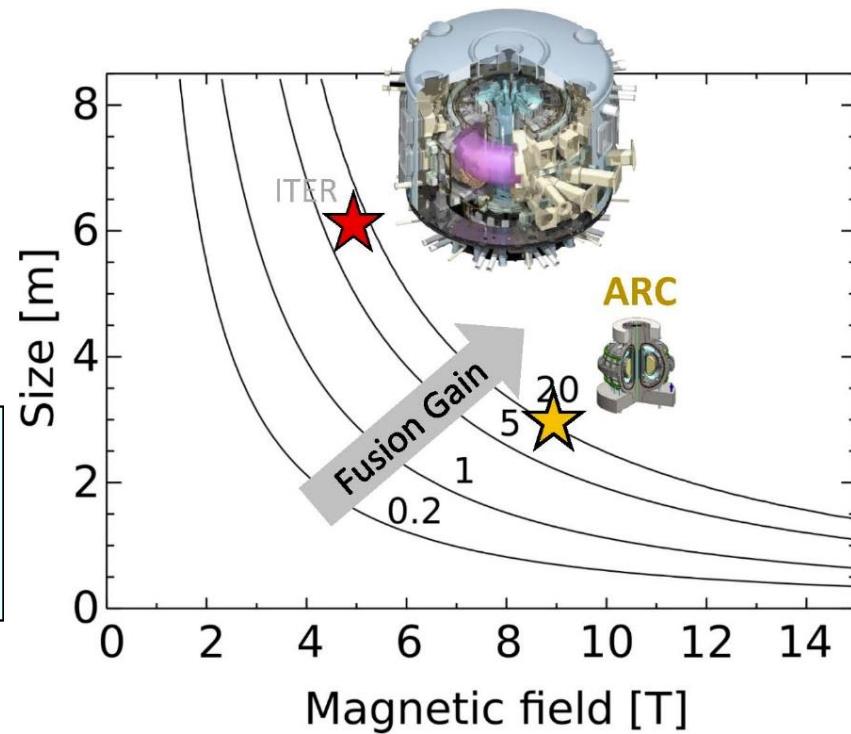


# ARC & SPARC @MIT

$$\frac{P_f}{V_P} \propto 8(p)^2 \propto \beta_T^2 B_0^4$$

$$nT \tau_E \sim \frac{\beta_N H}{q_*^2} R^{1.3} B^3$$

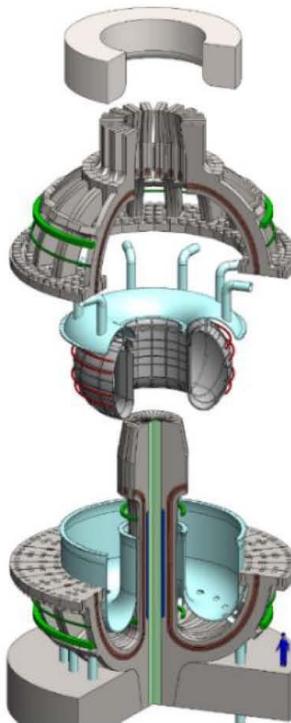
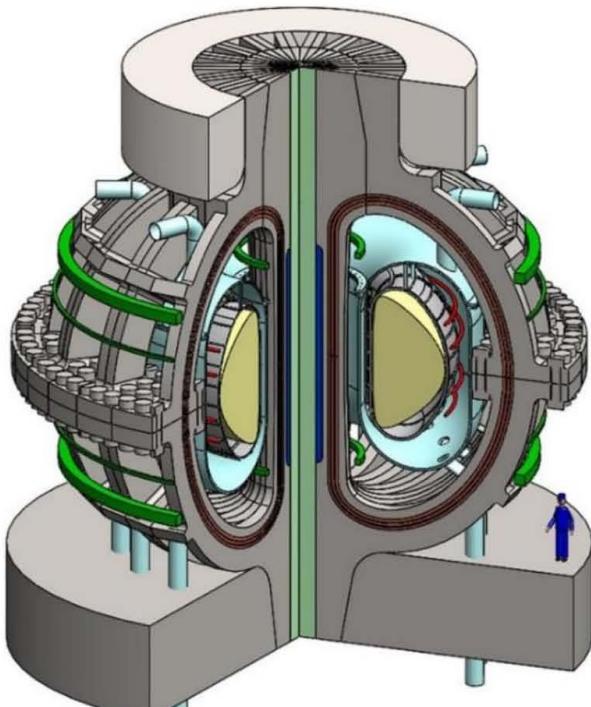
$$\frac{P_{fusion}}{S_{wall}} \sim \frac{\beta_N^2 \epsilon^2}{q_*^2} R B^4$$



**ARC**  
 $R = 3.3 \text{ m}$ ,  $B_t \sim 9 \text{ T}$ ,  $B_{\max} \sim 23 \text{ T}$   
 $P_f \sim 500 \text{ MW}$

ARC is a student-led fusion pilot plant concept that leverages high-field REBCO magnets to achieve numerous innovations at 10x smaller scale

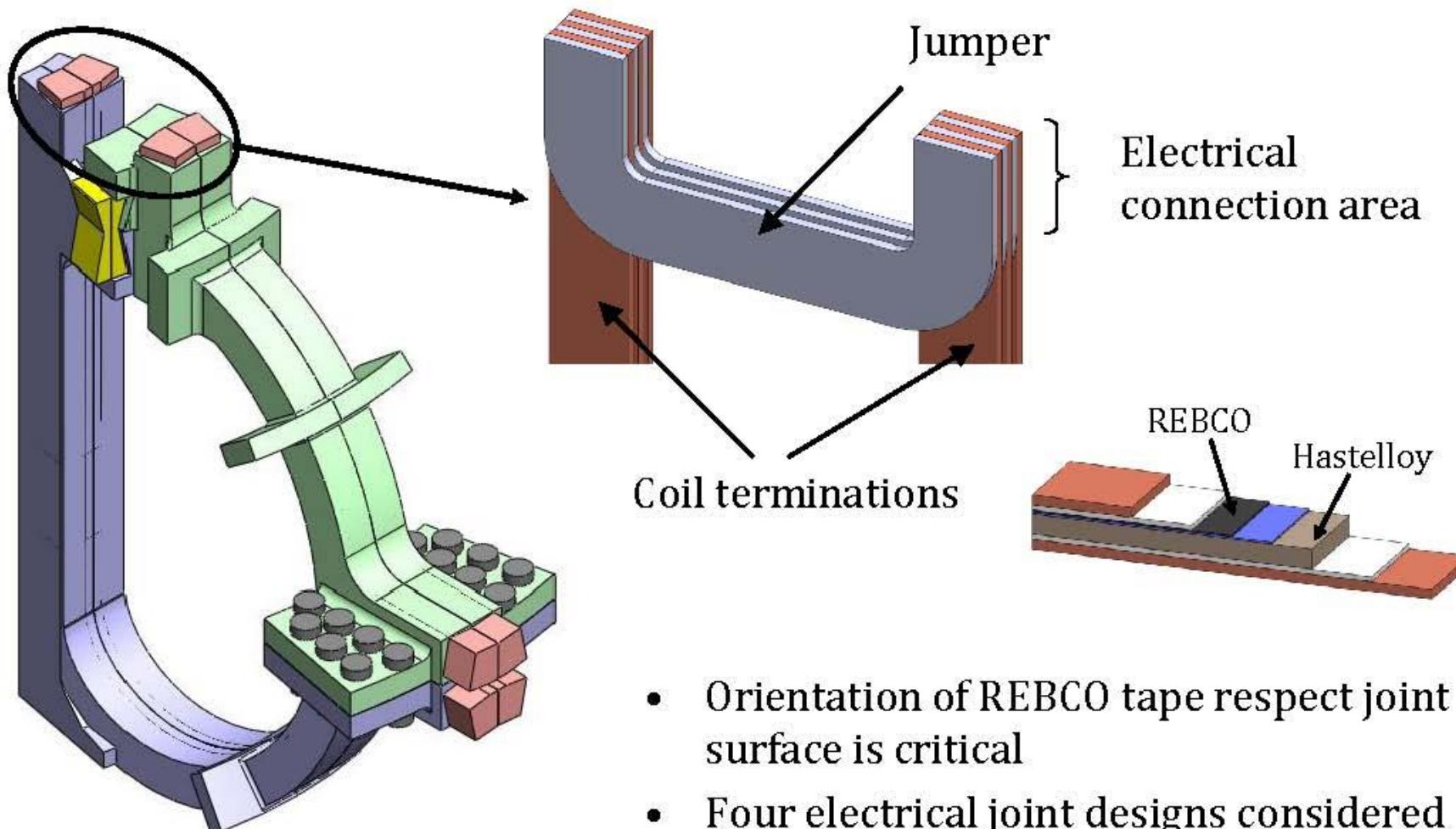
**PSFC**



B.N. Sorbom, "ARC: A compact, high-field fusion nuclear science facility and demonstration power plant with demountable magnets," *Fus. Eng. and Design* **100** (2015) 378.

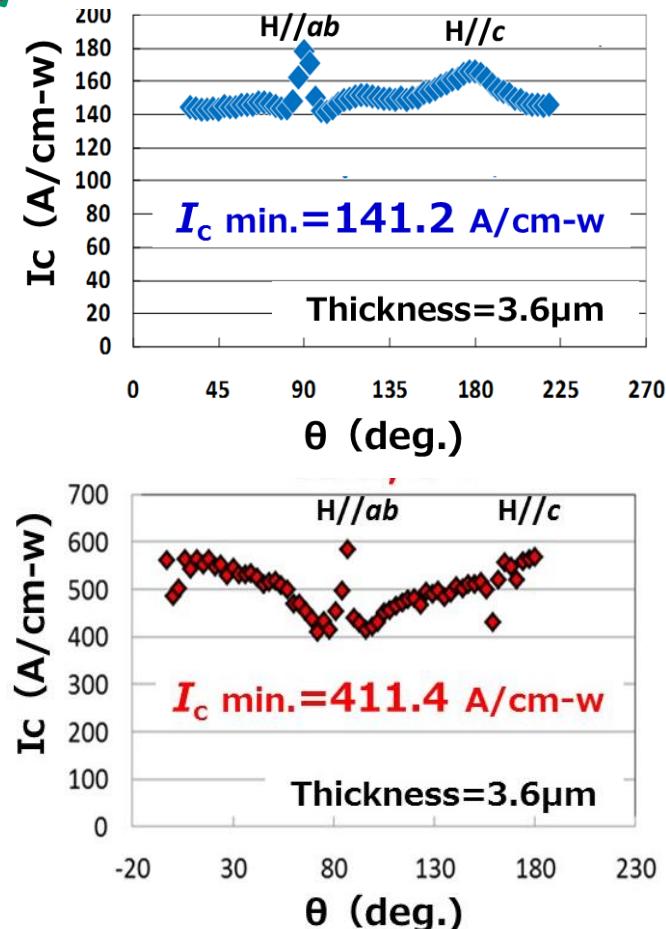
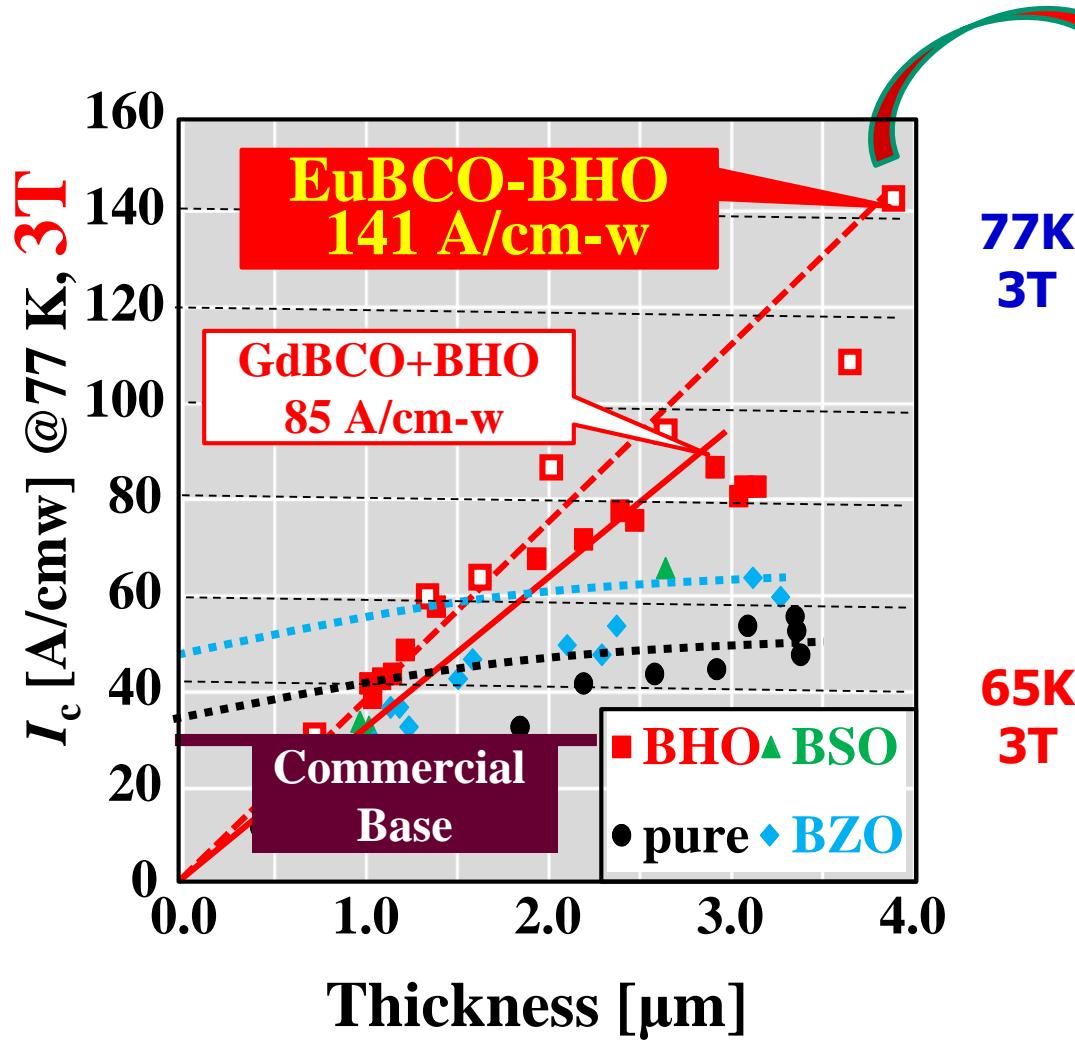
Design Parameters	Value
Fusion Power	500 MW
Total Thermal Power	700 MW
Conversion Efficiency	0.40 – 0.50
Net Electric Power	~200 MW
Plasma Gain, Q	>10
Major Radius, R	3.3 m
Inverse Aspect Ratio, $\epsilon$	0.34
Toroidal Field, $B_T$	9.2 T
Plasma Current, $I_p$	8 MA
Bootstrap Fraction	>60 %
Normalized Beta, $\beta_N$	2.5
Avg. Plasma Temperature, $\langle T_e \rangle$	14 keV
Avg. Plasma Density, $\langle n_e \rangle$	$1.75 \times 10^{20} \text{ m}^{-3}$
Tritium Breeding Ratio	1.10
Plant Lifetime	~10 FPY

## Electrical joint: resistive terminations linked with “jumper” plate



# Improvement of in-field $I_c$

## Effective Combination of REBCO & BMO(APC)



# **Extra Slides**

# HTS 導体を用いたヘリカル巻線に関する考察

- エッジワイス曲げ → フラットワイス曲げ + 捻りで代用
- ヘリカル巻線方向に対して捻りを調整  
エッジワイスひずみを最小限に抑える
- フラットな導体断面 → フラットワイスに曲げやすい

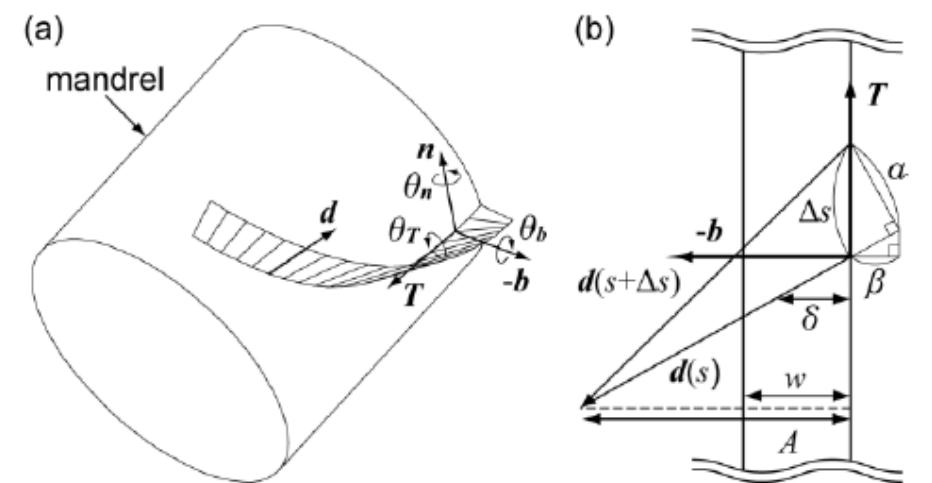
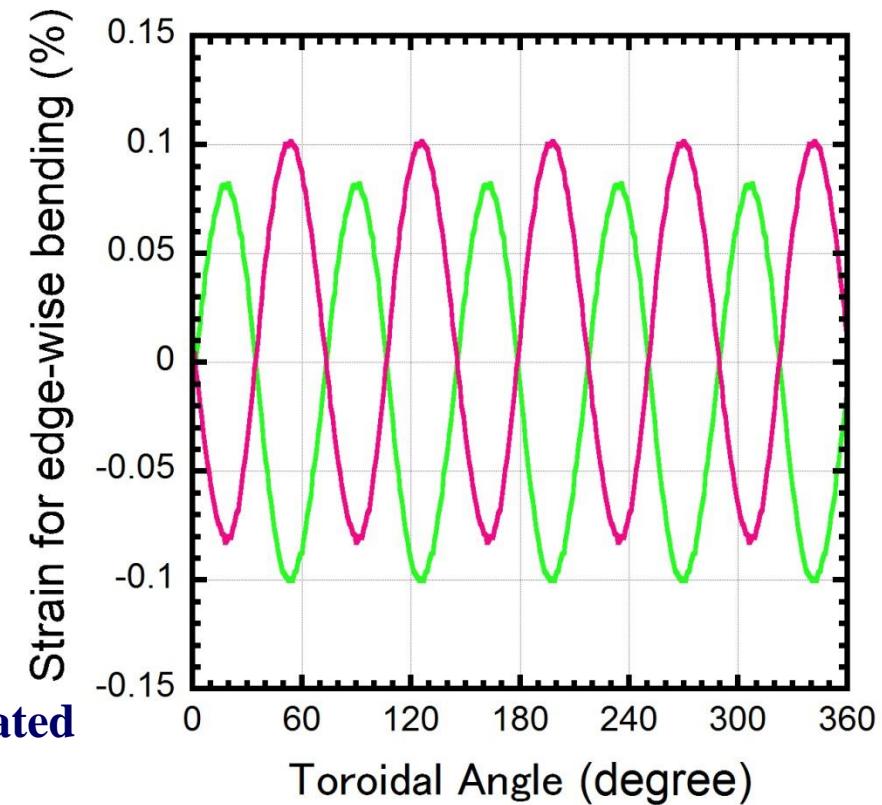
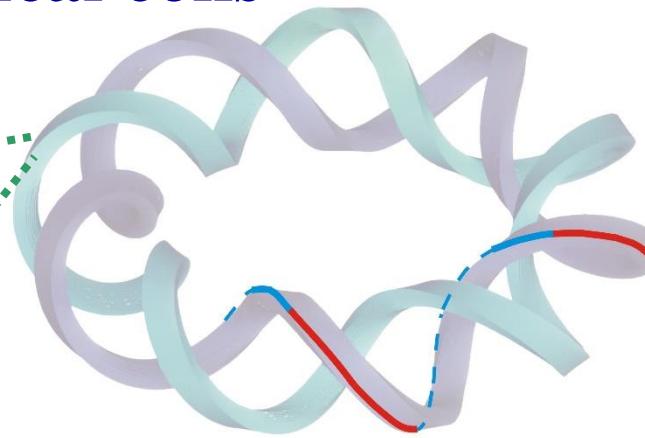
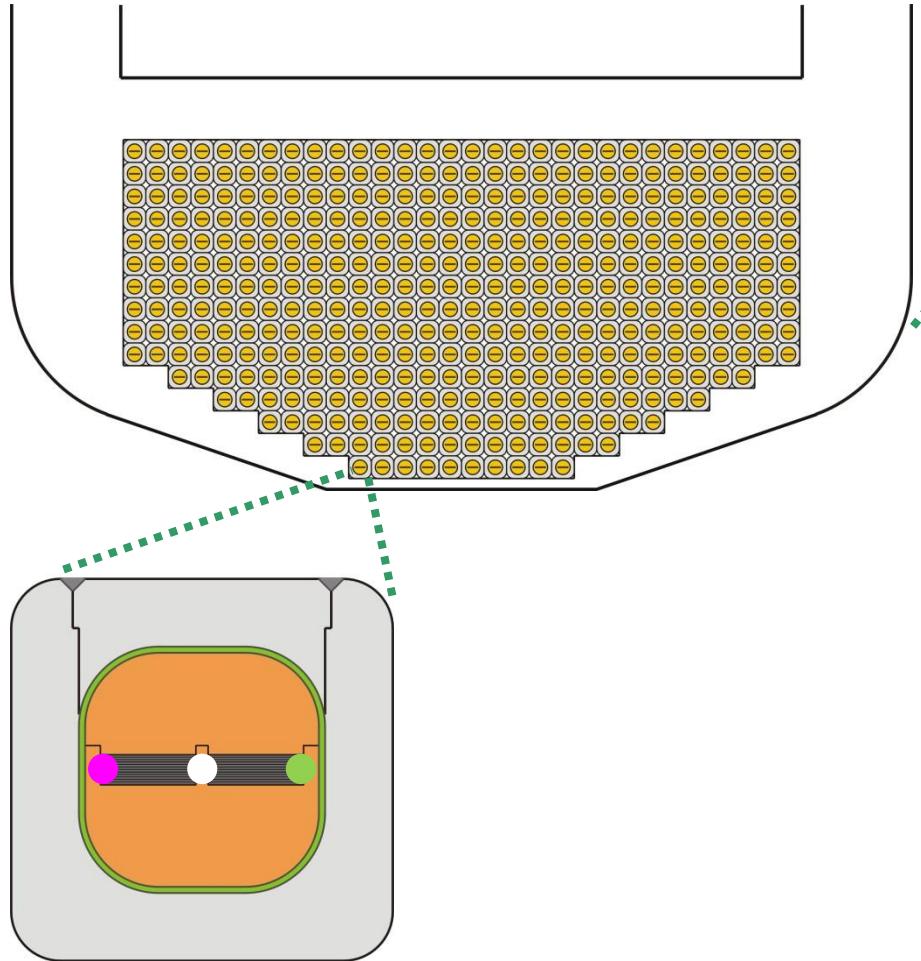


Fig. 2. (a) Conceptual view of three-dimensional winding. (b) Conceptual view of developed surface of tape.

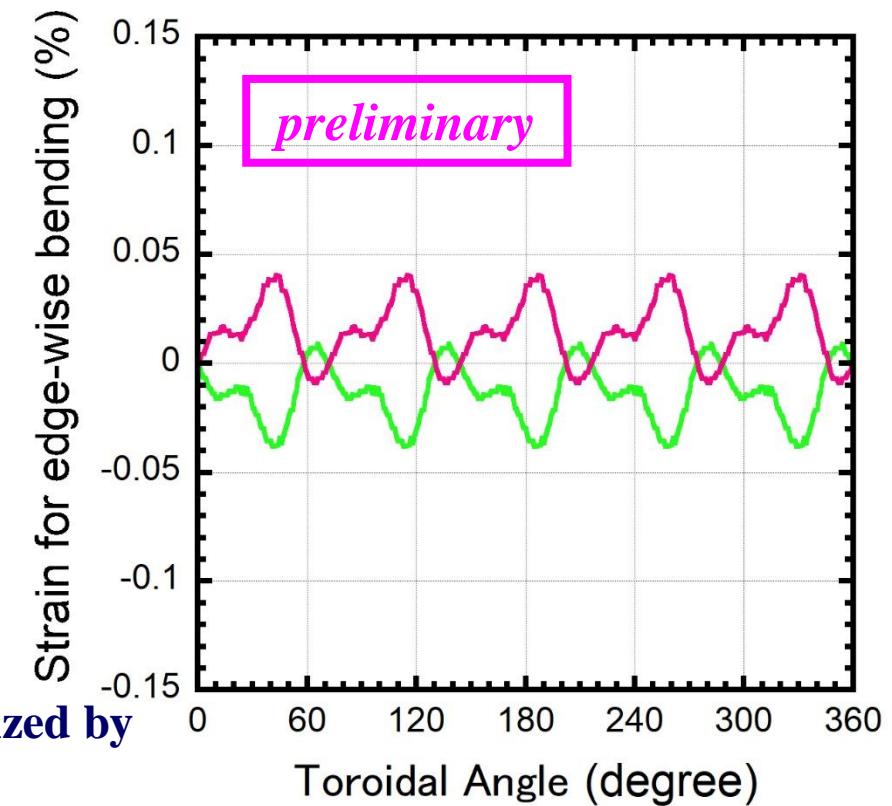
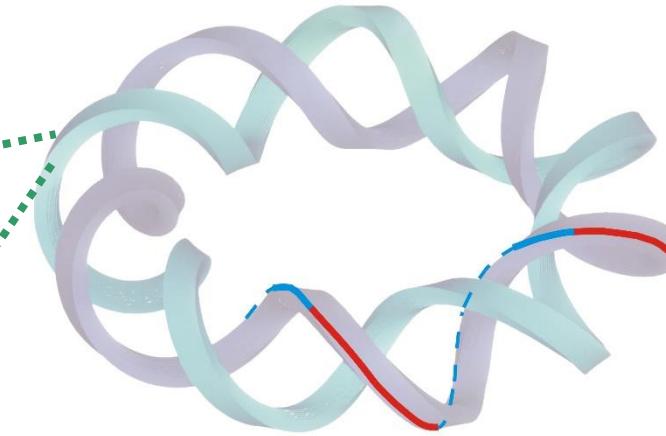
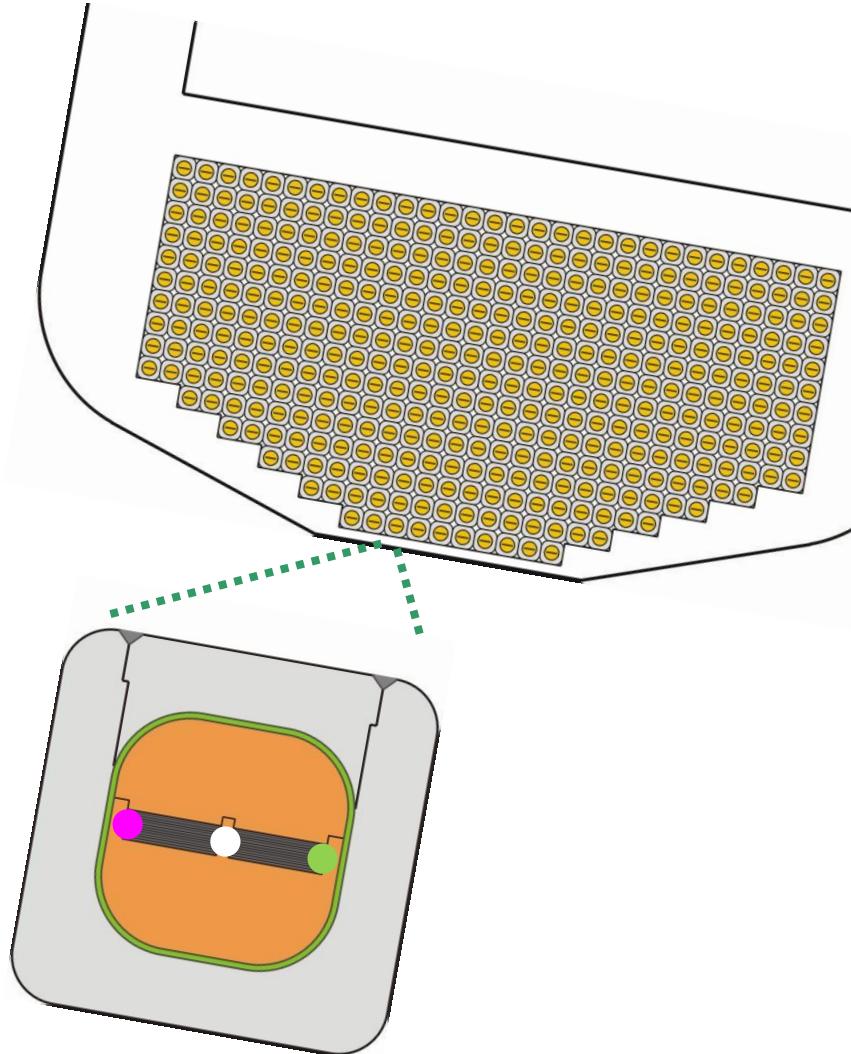
K. Takahashi, N. Amemiya, et al.,  
IEEE TAS 22 (2012)

# Edgewise strain on the HTS tape in the FFHR-d1 helical coils



- Difference of path length inside a conductor is calculated for evaluating edgewise strain

# Edgewise strain on the HTS tape in the FFHR-d1 helical coils



- Difference of path length (edge strain) can be minimized by tilting the whole winding package